



# NARSIS Workshop

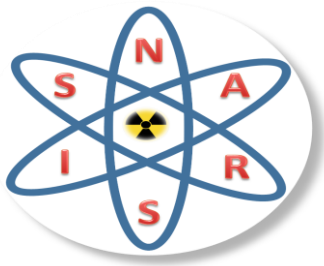
Training on Probabilistic Safety Assessment for Nuclear Facilities  
*September 2-5, 2019, Warsaw, Poland*



## Severe accident assessment with uncertainty and sensitivity analysis

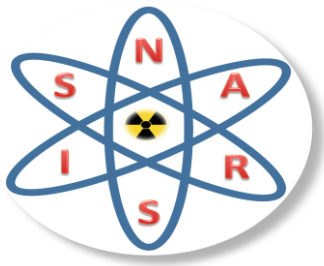
**Piotr Darnowski, Piotr Mazgaj**  
**Warsaw University of Technology**





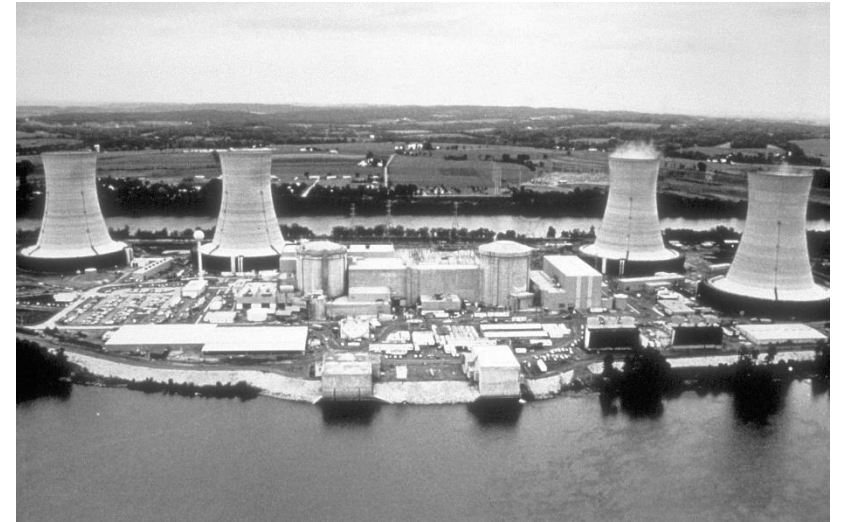
# Severe Accident Assessment

- **Prediction of NPP response to severe accident (SA)**
- **Types of the severe accident tools:**
  - ❑ Integral codes: MELCOR, MAAP, ASTEC.
  - ❑ Specific phenomena codes: ICARE/CATHARE, ATHLET\_CD, RELAP/SCDAP, CONTAIN, COCOSYS
  - ❑ Specific integral codes sets: SAMPSON, RELAP/SCDAP – CONTAIN – VICTORIA, ATHLET-CD – COCOSYS.
  - ❑ CFD are used for some detailed phenomena.
- **Level of details**
  - ❑ Engineering Codes (i.e. MAAP)
  - ❑ Semi-Mechanistic Codes (MELCOR, ASTEC)
  - ❑ Mechanistic Codes (i.e. RELAP/SCDAPSIM)
  - ❑ CFD (i.e. OpenFoam)



# SA Assessment with Integral Codes

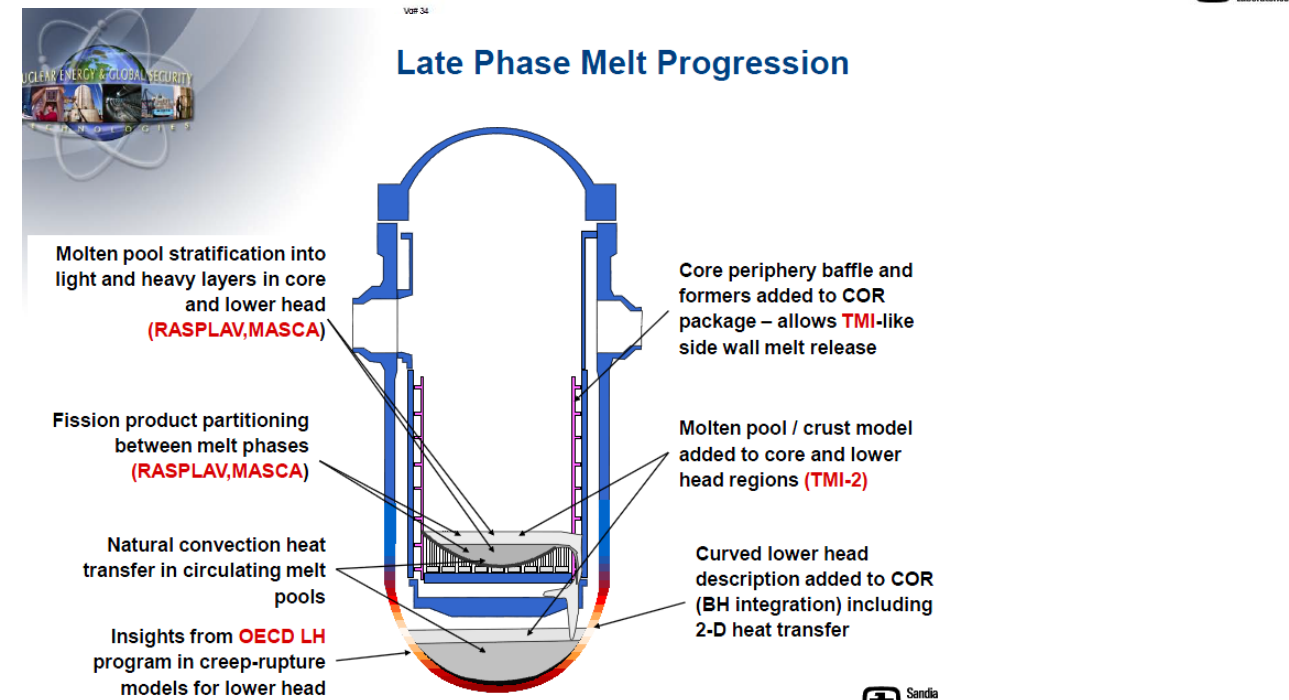
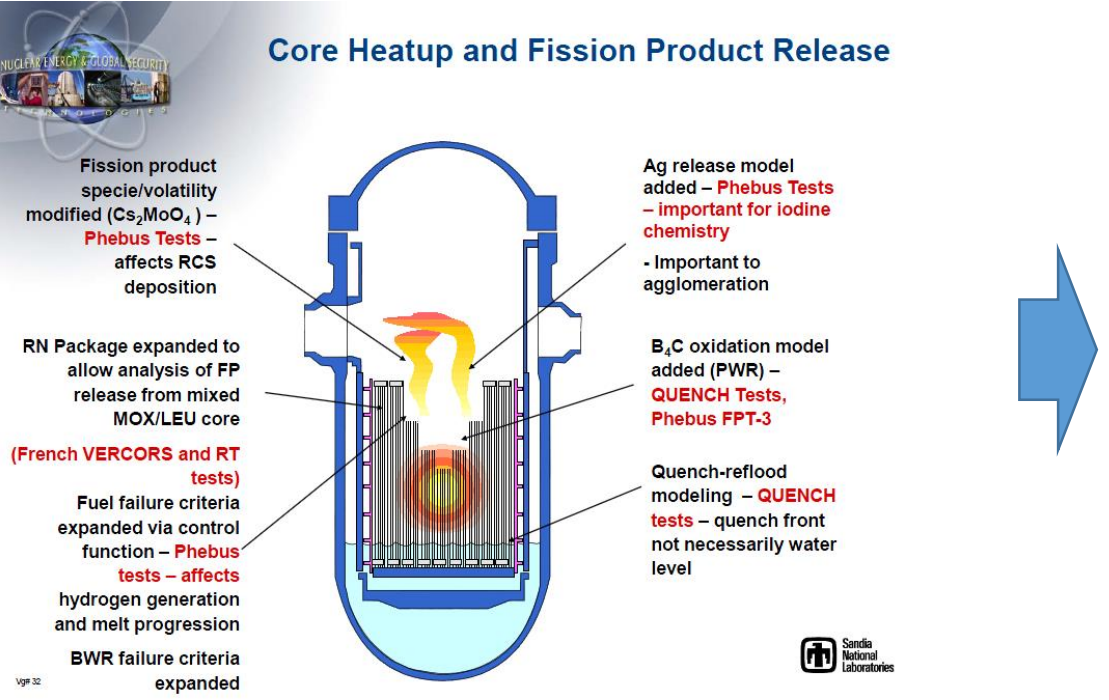
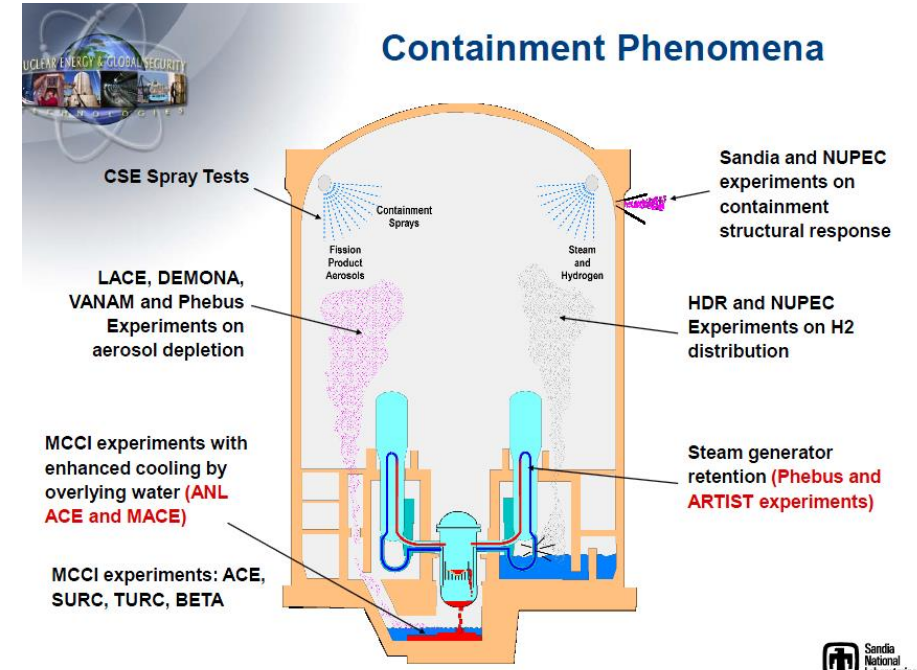
- **Integral codes allow to simulate whole plant**
- **Typical goals:**
  - Accident progression
  - Containment performance
  - Source term estimation
- **Part of PSA Level 2**
- **Input for PSA Level 3**
- **Support of SA management, staff training etc.**
- **The most popular codes:**
  - ASTEC
  - MAAP
  - MELCOR





# SA Integral Codes

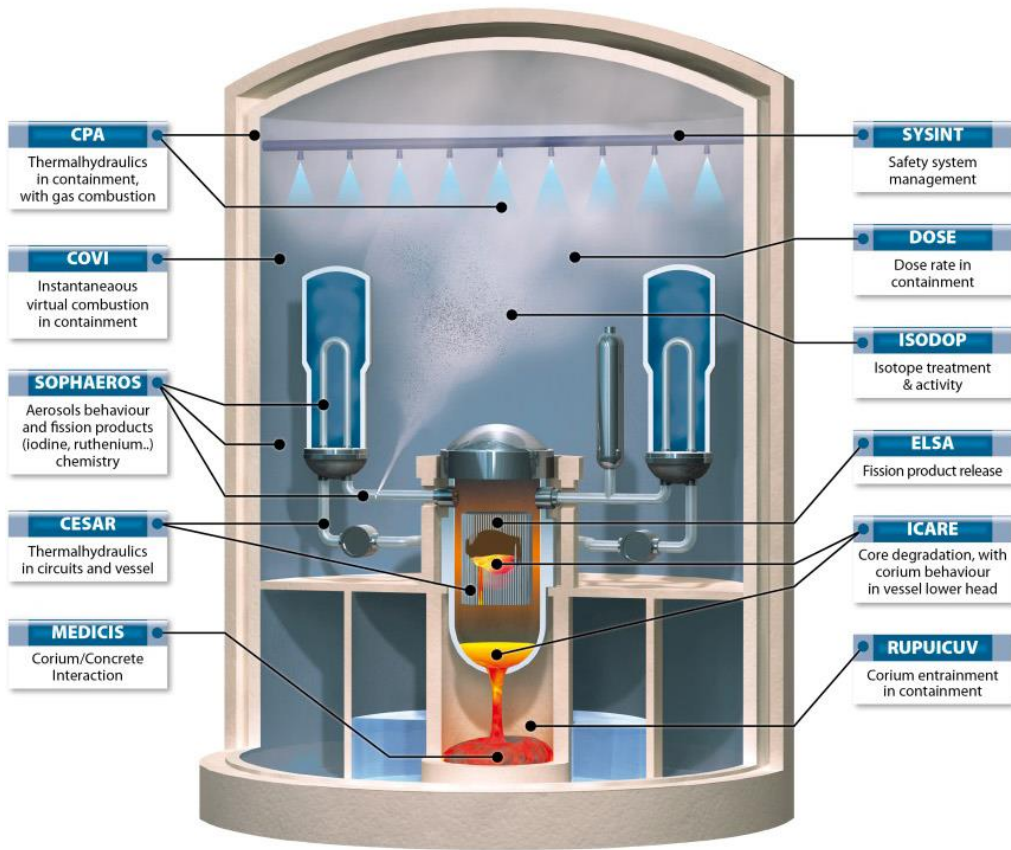
- Integral codes simulate almost all relevant phenomena
- Large experimental data portfolio
- Codes validated and verified (V&V)
  - ❑ MAAP, MELCOR >30 years
  - ❑ ASTEC >20 years





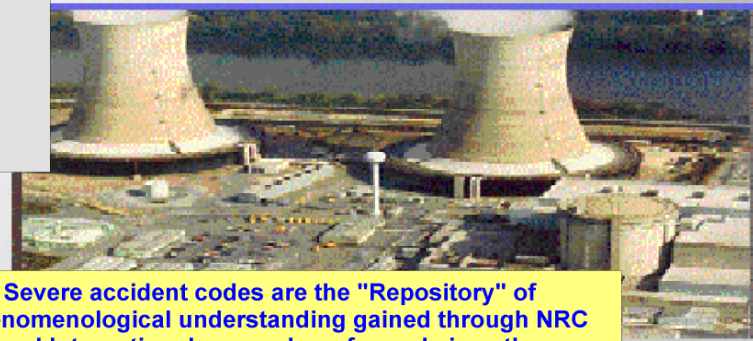
# SA Integral Codes

## ASTEC



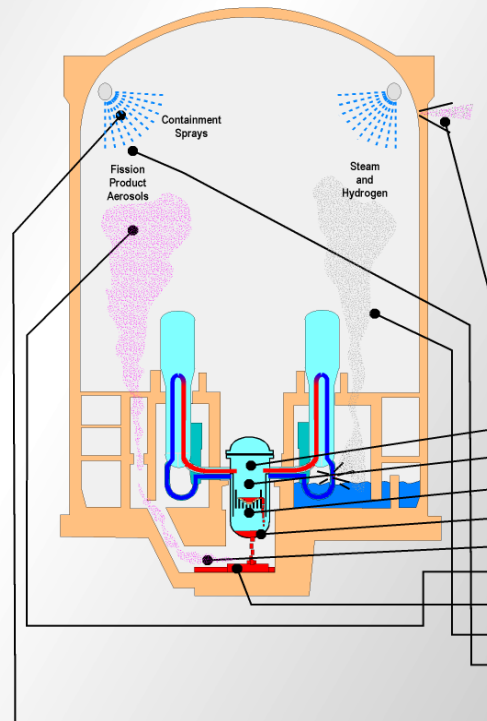
## MELCOR

### Modeling and Analysis of Severe Accidents in Nuclear Power Plants



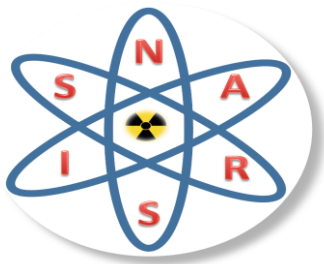
Severe accident codes are the "Repository" of phenomenological understanding gained through NRC and International research performed since the TMI-2 accident in 1979

*Integrated models required for self consistent analysis*



#### Important Severe Accident Phenomena

Phenomenon	MELCOR	CONTAIN	VICTORIA	SCDAP	RELAP 5
Accident initiation	█				
Reactor coolant thermal hydraulics	█				
Loss of core coolant	█				
Core meltdown and fission product release	█		█		
Reactor vessel failure	█				
Transport of fission products in RCS and Containment	█	█	█		
Fission product aerosol dynamics	█	█	█		
Molten core/basemat interactions	█				
Containment thermal hydraulics	█	█			
Fission product removal processes	█	█	█		
Release of fission products to environment	█	█	█		
Engineered safety systems - sprays, fan coolers, etc iodine chemistry, and more	█	█			



# Modelling Experiments

## ➤ Experimental facilities

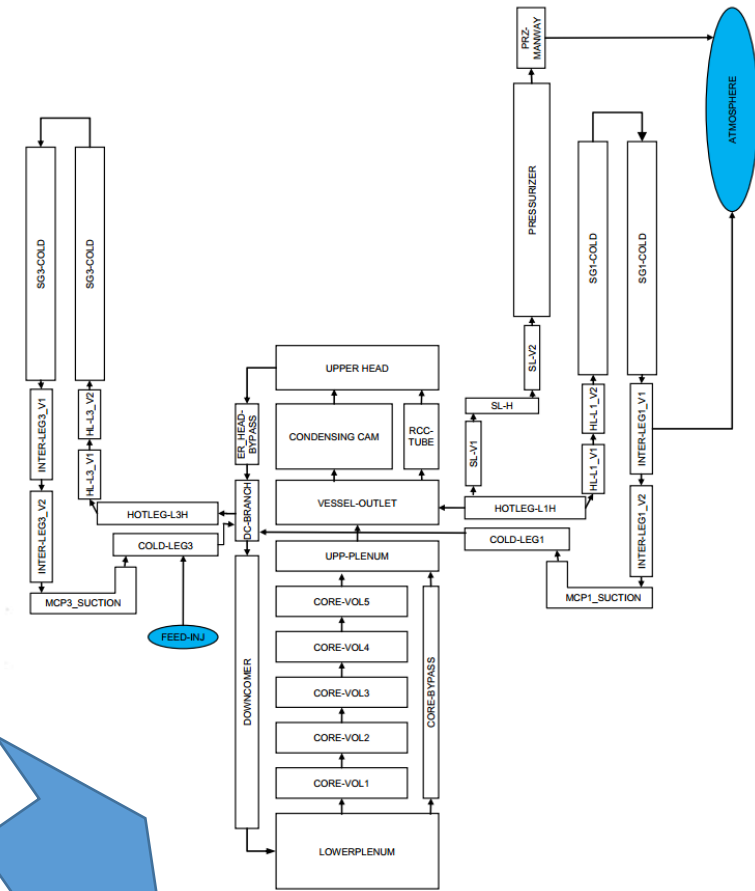
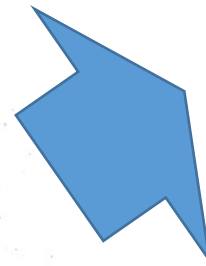
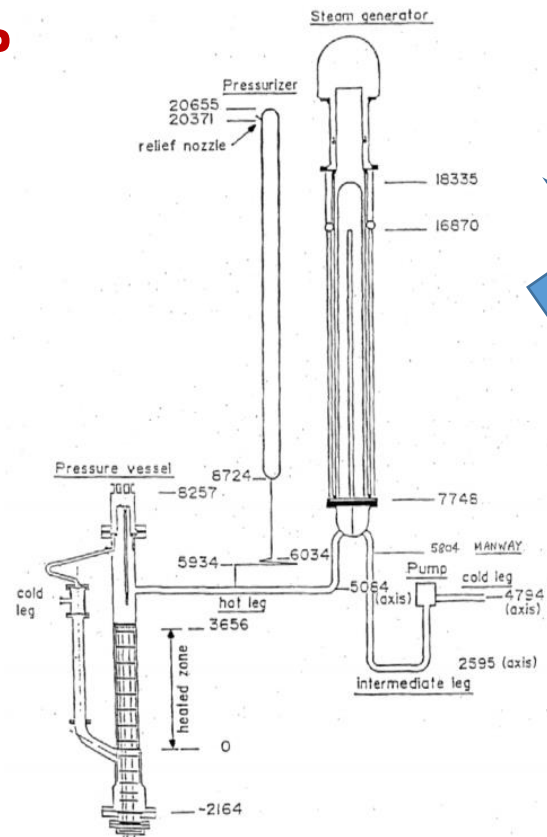
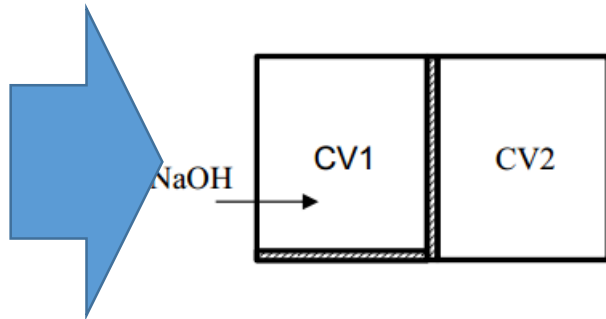
- ❑ Separate Effect Test
- ❑ Integral Experiments

## ➤ Usually we start with experiments

## ➤ Apply gained experience to simulate NPP



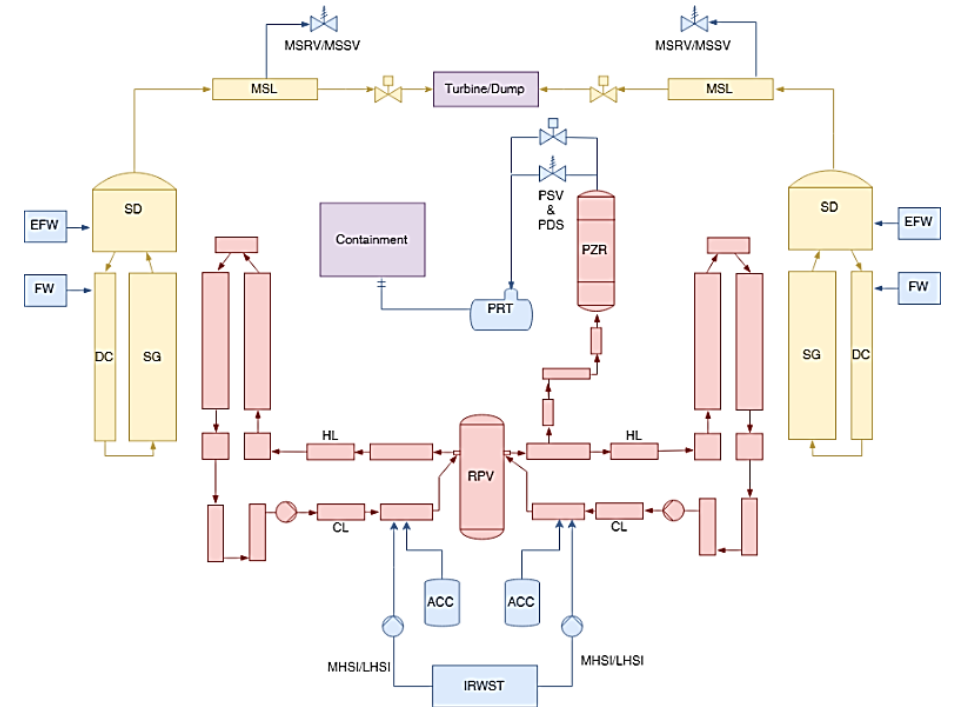
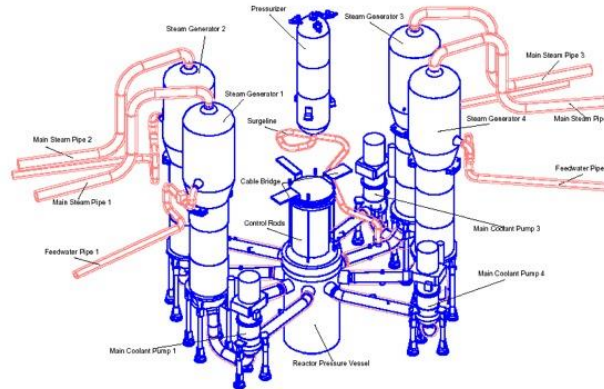
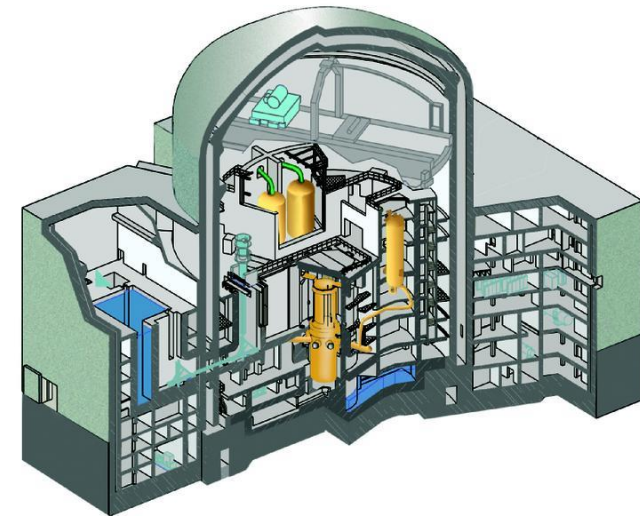
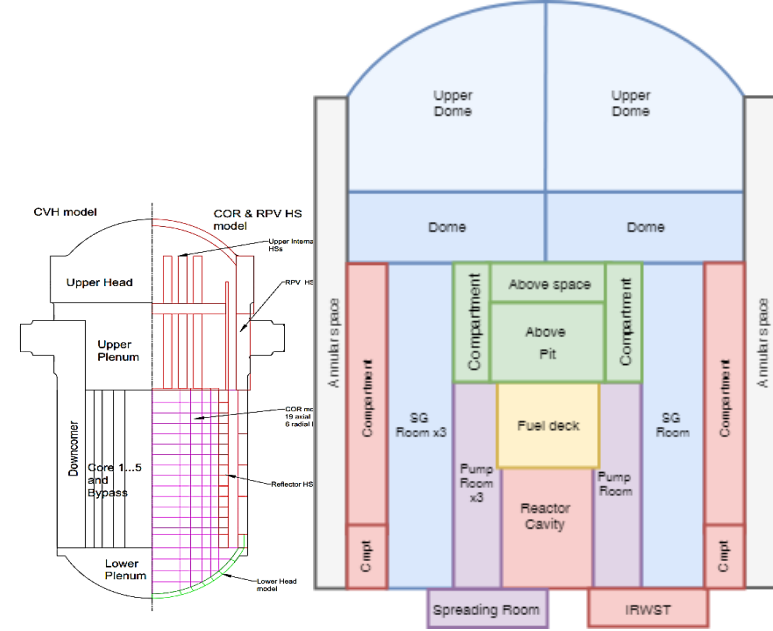
TRANSIT WORKSHOP

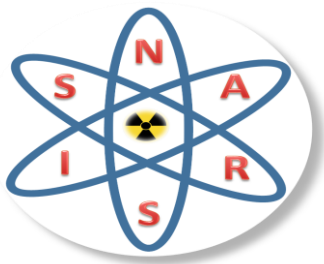




# Modelling NPP

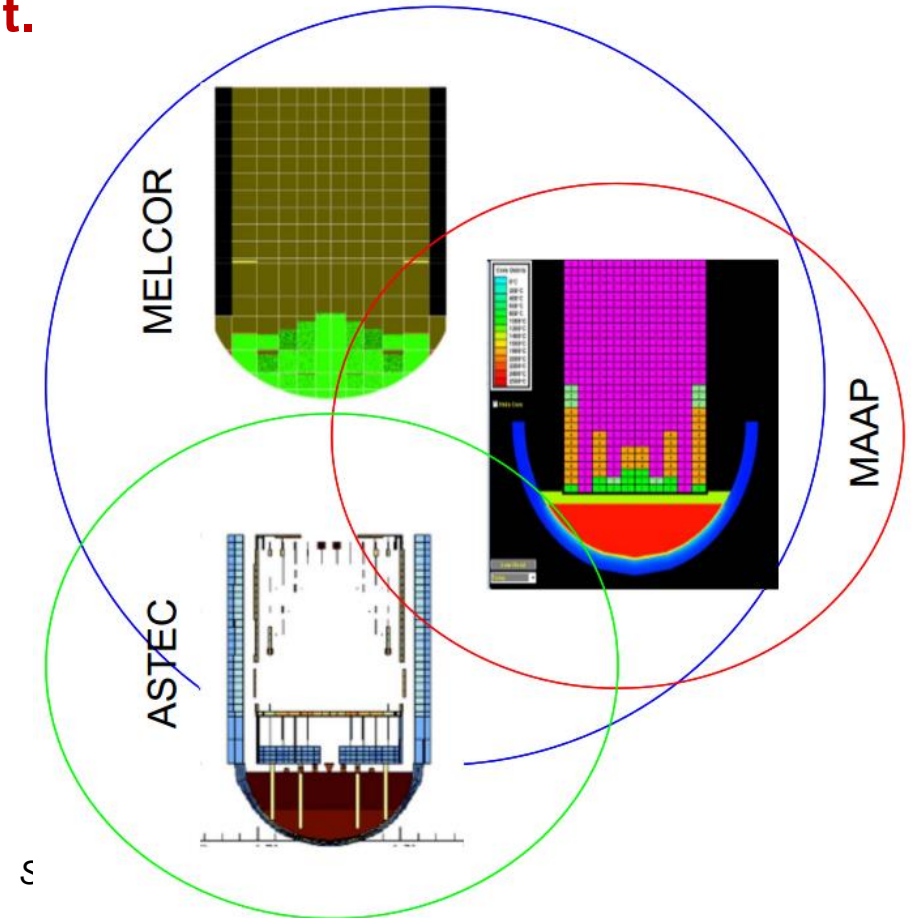
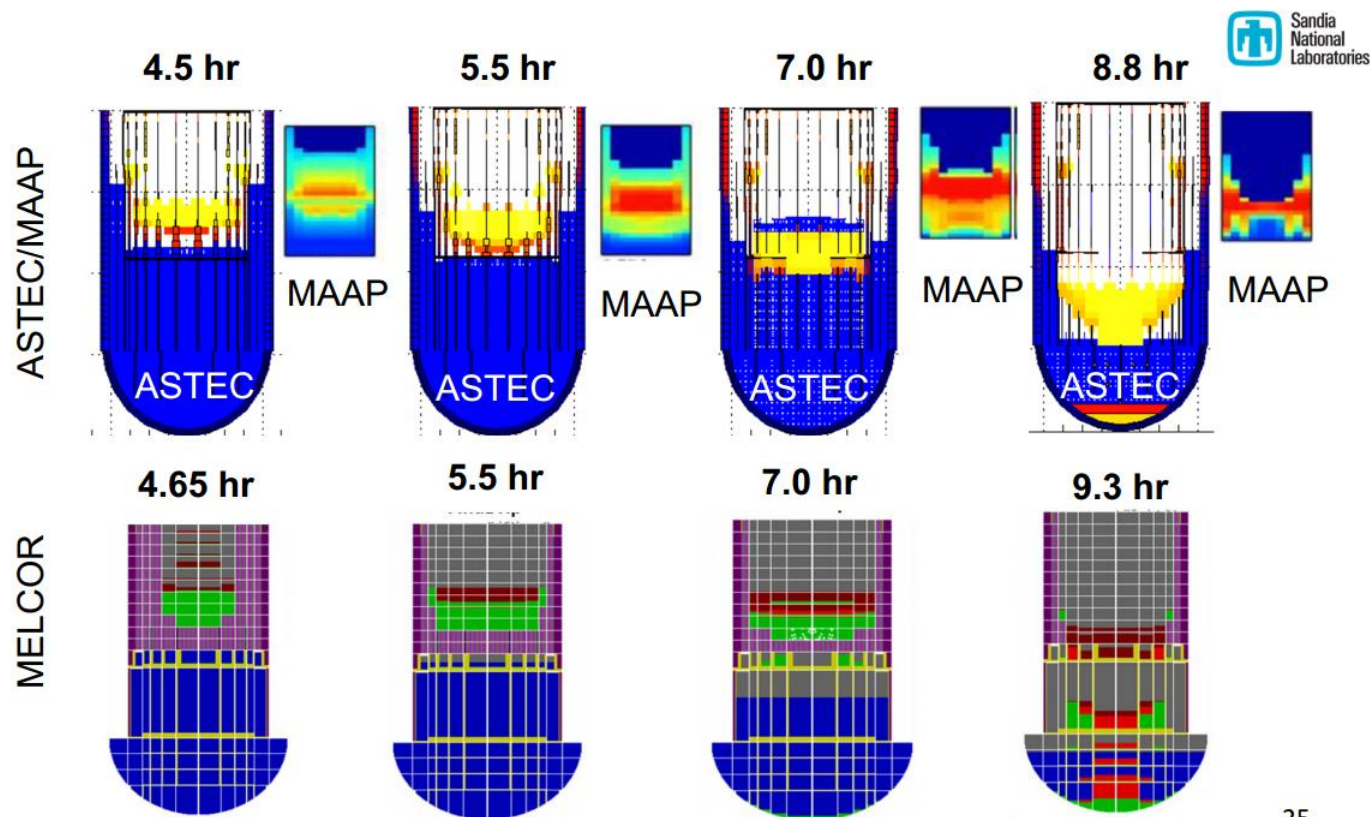
- Plant models
- Successful, experiment simulations does not guarantee successful plant simulations
- Large knowledge and experience is necessary



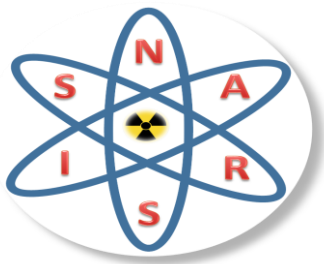


# SA Integral Simulations

- Different codes can provide different results.
- Different users can obtain different results with the same code (user effect).
- Code cross comparison and benchmarks are important.

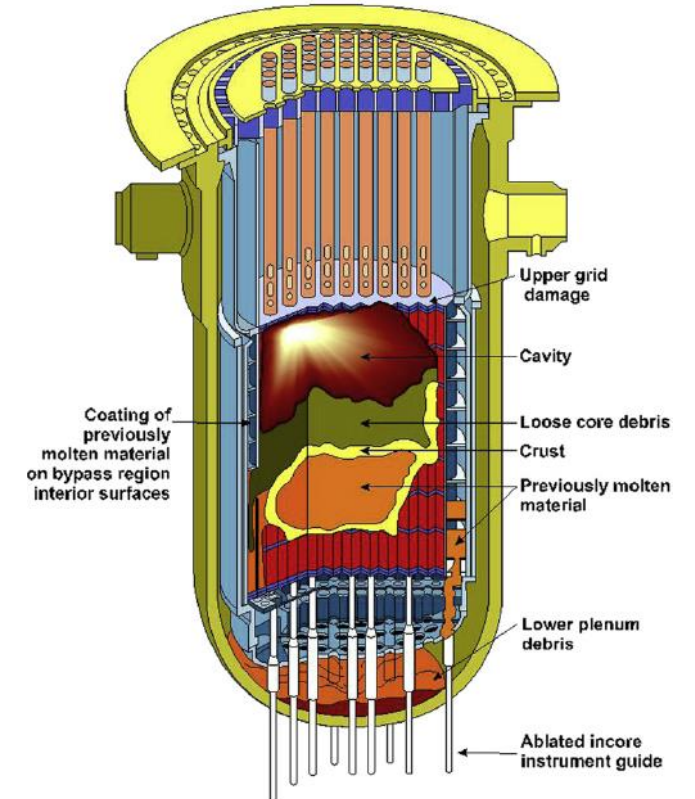
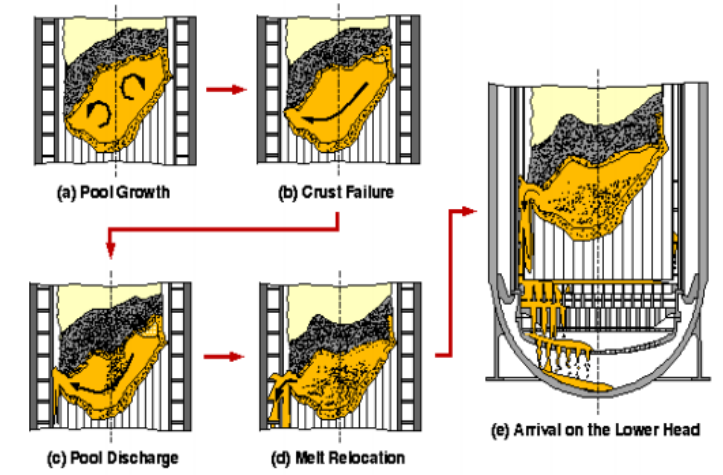


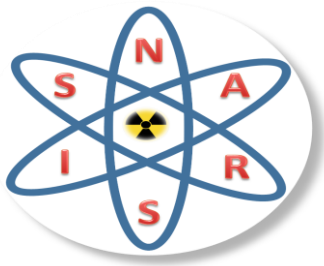




# Uncertainties in SA

- **NPP is a complex system**
- **Severe accident phenomenology is also complex**
- **Uncertainties are unavoidable**
- **Experimental data uncertainties**
- **Models uncertainties**
- **Various factors influence SA assessment**
  - ❑ **Variability; systems, human factors, other**
  - ❑ **Lack of knowledge about details of the phenomena**
  - ❑ **In principle relevant phenomena are recognized**
  - ❑ **Modelling precision, discretization (nodalization)**
  - ❑ **Codes, user effects, modeling approach**
- **Uncertainty qualification is unavoidable**

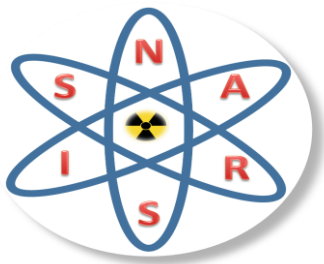




# Sensitivity and Uncertainty Analysis (S&UA)

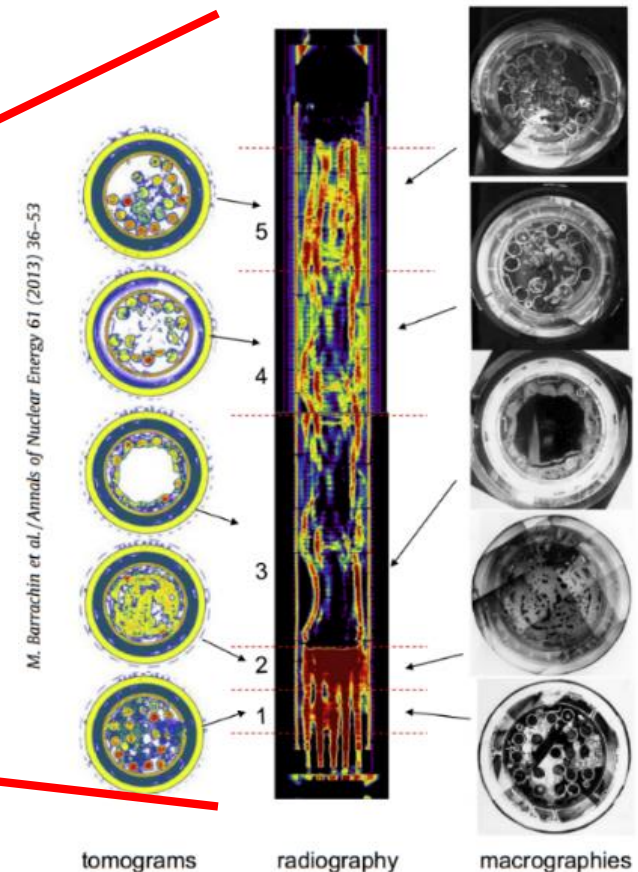
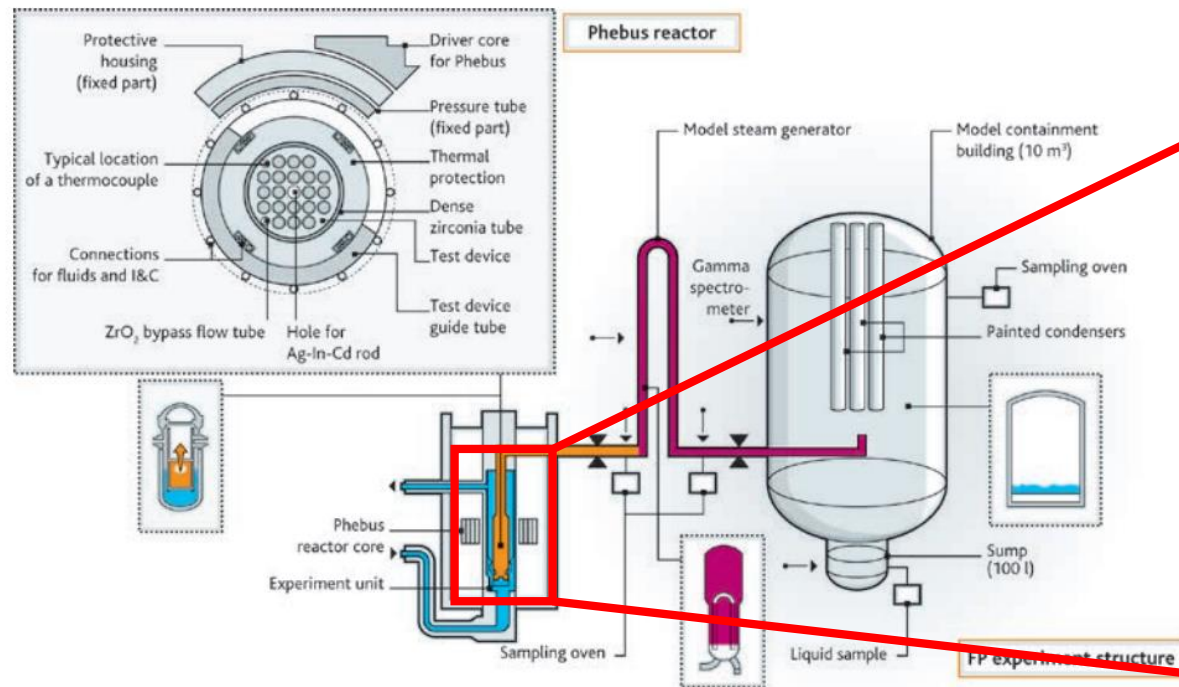
## ➤ Typical S&UA in NPP

1. Identification of uncertain input variables/models
2. Assignment of uncertainty information (distributions)
3. Determination of the sample size for the statistical significance of the uncertainty measures for the output variables.
4. Sampling.
5. Code execution
6. Post-processing of results.
7. Statistical analysis. Uncertainty and Sensitivity quantification.
8. Study of individual cases/outliers.



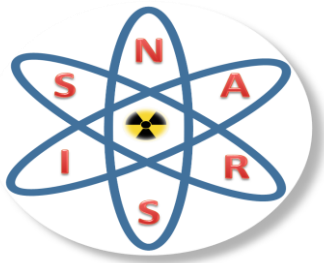
# Example – Hydrogen source term in FPT-1

- Example MELCOR application - FPT-1 integral experiment.
- H<sub>2</sub> source term in Phebus FPT-1 test.
- Basically: How much hydrogen was generated during the core degradation ?
- Uncertainty and sensitivity analysis.



M. Barrachin et al. / Annals of Nuclear Energy 61 (2013) 36–53

Fig. 4. FPT1, non-destructive and destructive examinations of the bundle.



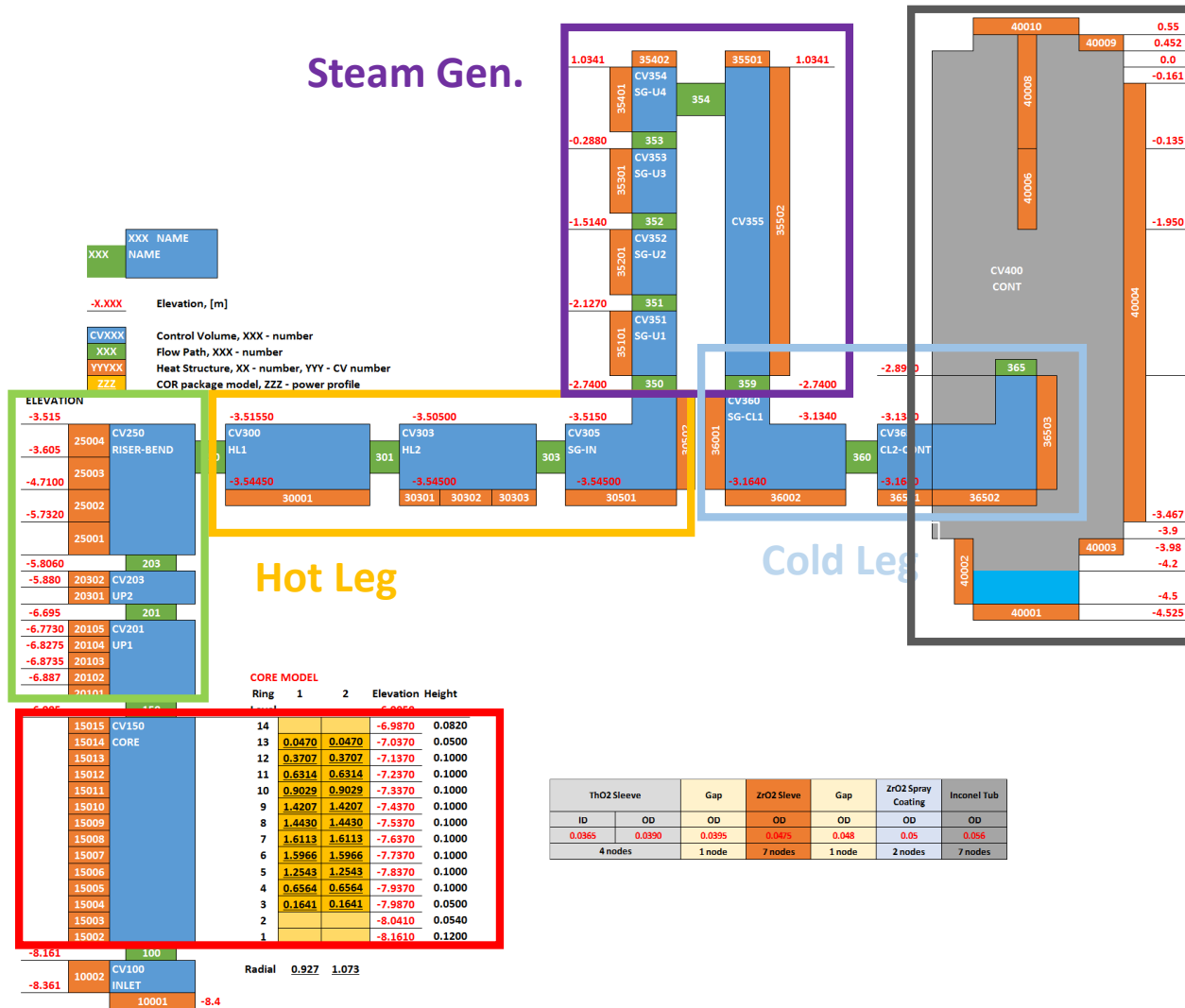
# Example - Model Development

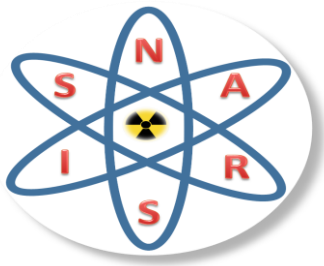
Upper plenum

Core model

Steam Gen.

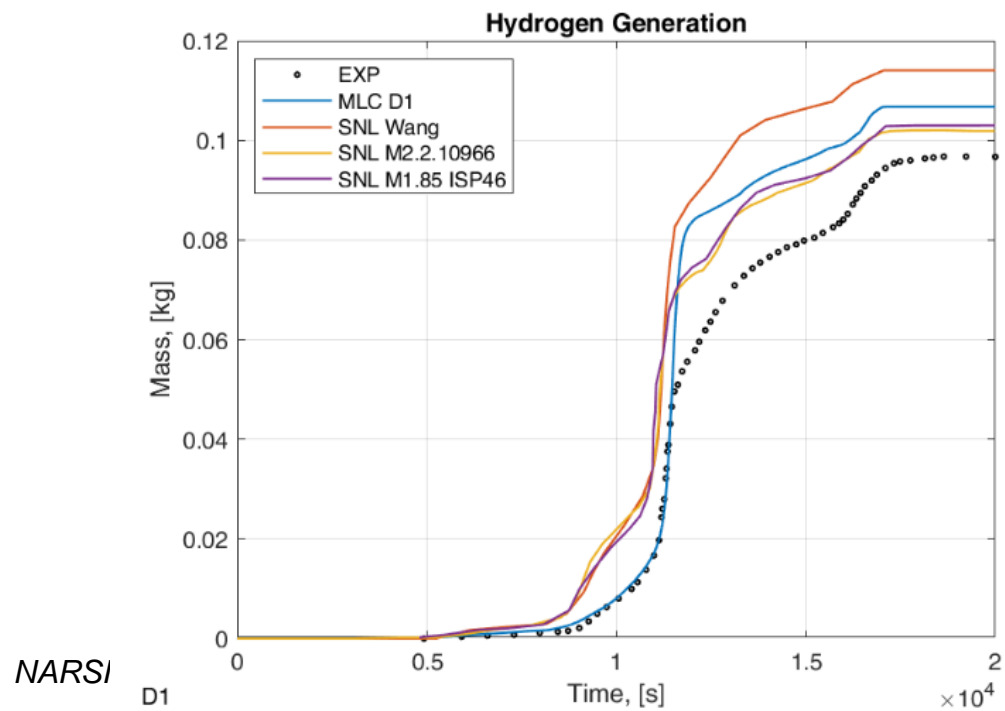
Containment



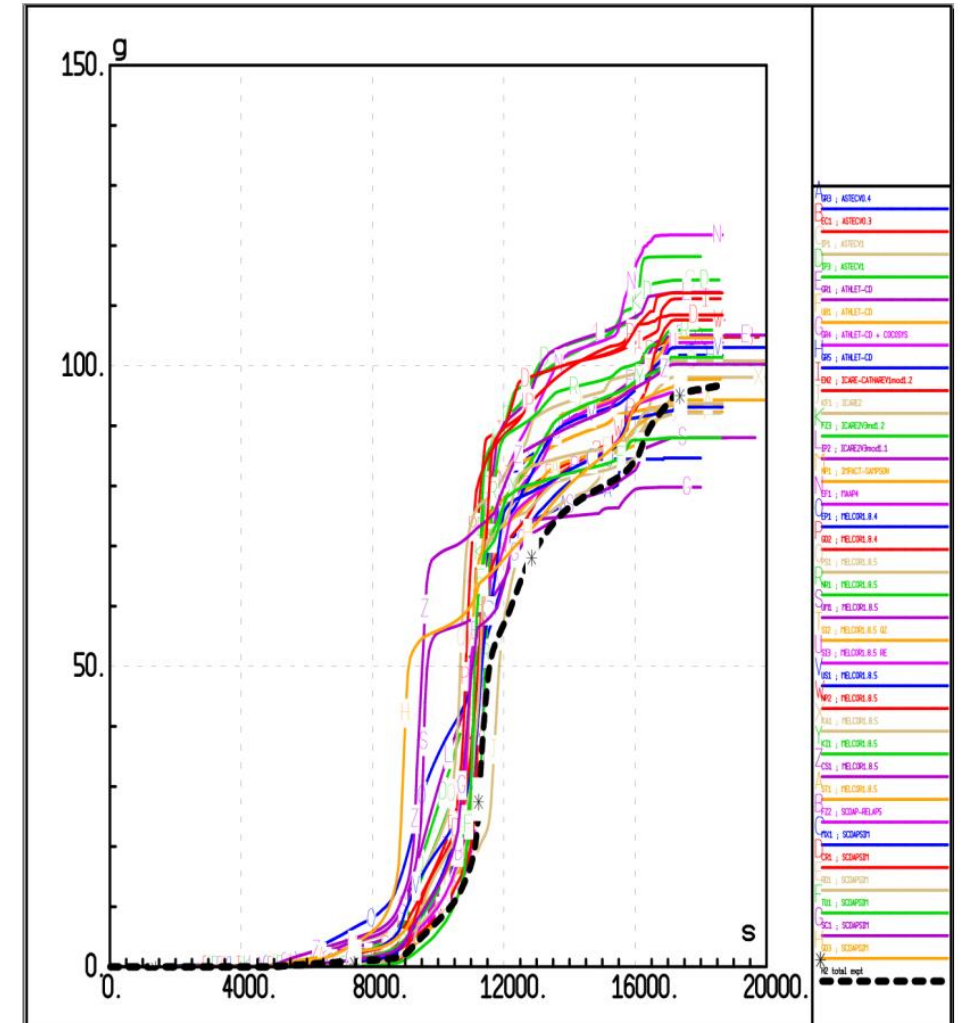


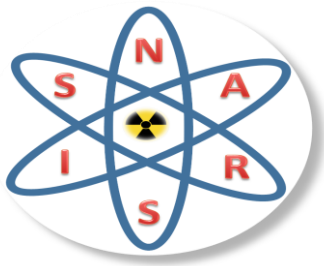
# Example – Tests and Simulations

- Use state-of-the art modeling – best estimate calculation
- Comparison with experimental data (if available).
- Comparison with the literature (if possible).
- Comparison with other codes (if possible).



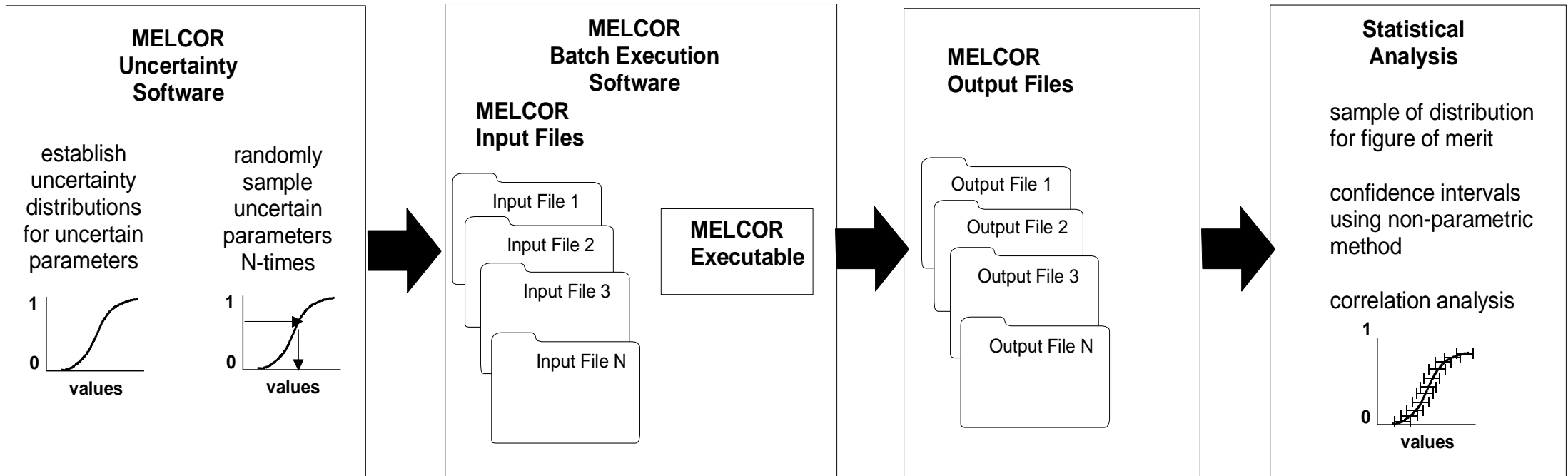
## International Standard Problem 46 Report results





# Example – S&UA Methodology

## ➤ Sensitivity and Uncertainty Methodology (with MELCOR)

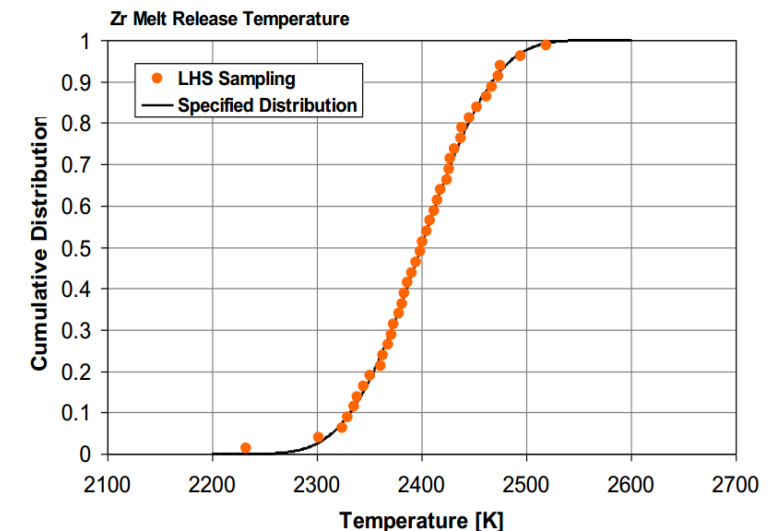
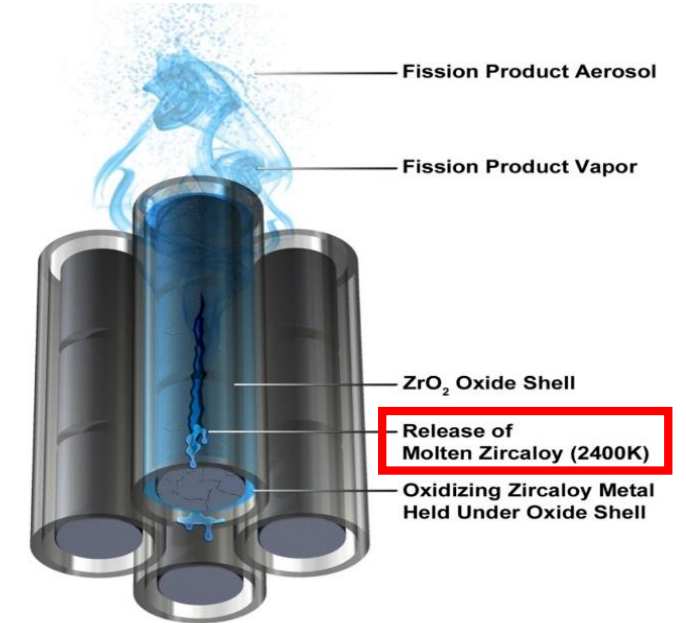
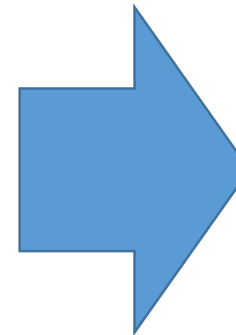


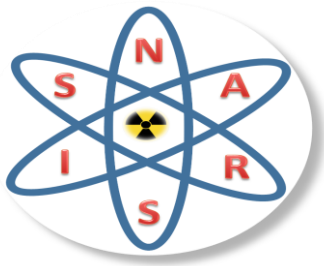


# Example - Identification of uncertainties

- Identify uncertain input variables or models
- Identify probability distributions
- Example - simple as there is Gantt report Ref. [3].
  - ☐ Parameter #1 Zr melt breakout temperaute

No	Parameter	Probability distribution
0	Oxidation Rate Coefficients	Discrete, Uniform
<b>1</b>	<b>Molten Material Holdup Parameters - Zr Melt Breakout Temperature</b>	<b>Normal</b>
2	Core (Fuel) Component Failure Parameters - Fuel Rod Failure Temperature	Normal
3	Secondary Material Transport Parameters - Secondary UO2 Content	Normal
4	Candling Heat Transfer Coefficient - Zr Freezing	Log-Normal
5	Core-Region Particulate Debris Diameter	Log-Normal
6	Debris porosity	Triangular
7	Radiation Exchange Factor Radial	Normal
8	Radiation Exchange Factor Axial	Normal
9	Molten clad drainage rate	Log-Normal





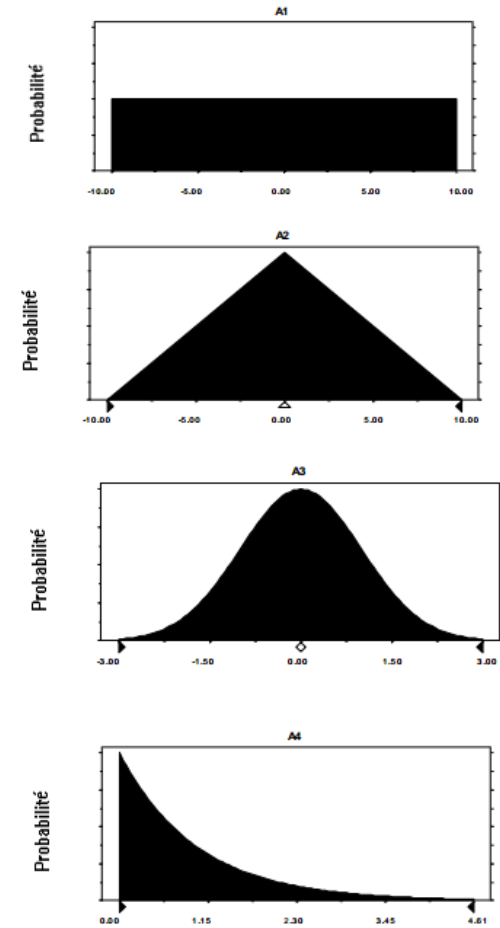
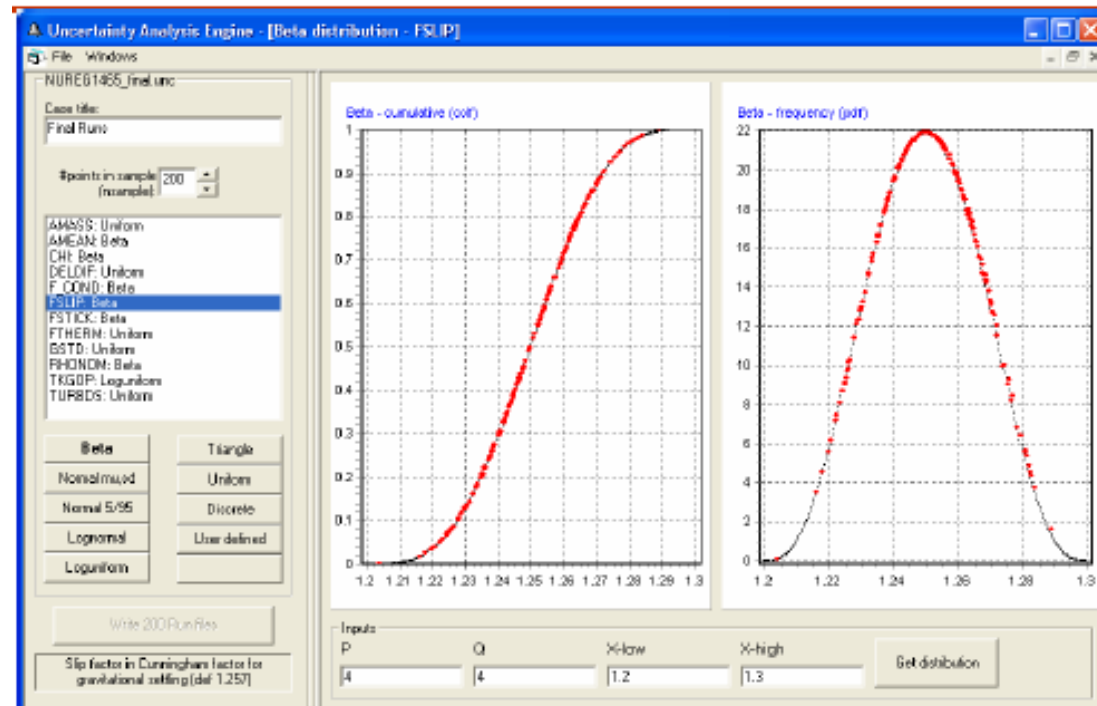
# Identification of uncertainties and distributions

## ➤ Codes have dedicated tools to sample and pre-process and post-process

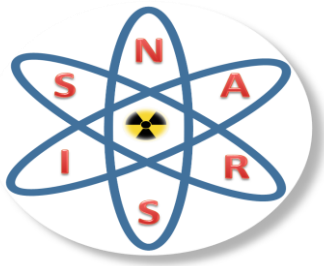
- MELCOR Uncertainty Engine
- ASTEC SUNSET Package

## ➤ Typical distributions (i.e. for MELCOR)

- Uniform
- Triangular
- Normal
- Exponential
- Log-normal
- Log-triangular
- Log-uniform
- Beta
- Discrete







# Sample Size - Samuel Wilks

- Samuel Wilks
- S. S. Wilks, “Determination of Sample Sizes for Setting Tolerance Limits,” *Ann. Math. Stat.*, vol. 12, no. 1, pp. 91–96, 1941.

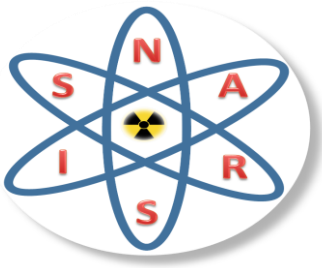


## DETERMINATION OF SAMPLE SIZES FOR SETTING TOLERANCE LIMITS

By S. S. WILKS

*Princeton University, Princeton, N. J.*

**1. Introduction.** In the mass production of a given product or apparatus piece-part, Shewhart<sup>1</sup> has discussed a practical procedure for detecting the existence of assignable causes of variation in a given quality characteristic of the product as measured by a variable  $x$ . For example,  $x$  may be the thickness in inches of a washer or the tensile strength in pounds of a small aluminum casting made according to a given set of specifications;  $x$  varies in value from washer



## Example – Sample Size

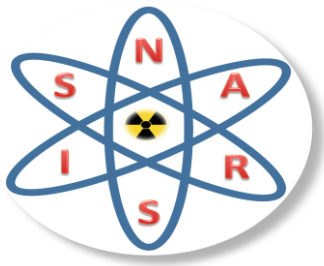
### ➤ **Select size of the sample.**

- ❑ It takes ~3h per one run
- ❑ Limited computational resource and more simulation can be too expensive
- ❑ Popular approach – Wilks formula
- ❑ Example - 93 samples 95%/95%

### ➤ **More samples – greater % of distribution to be sample with higher confidence**

$$C = 1 - n \cdot p^{n-1} + (n + 1) \cdot p^n$$

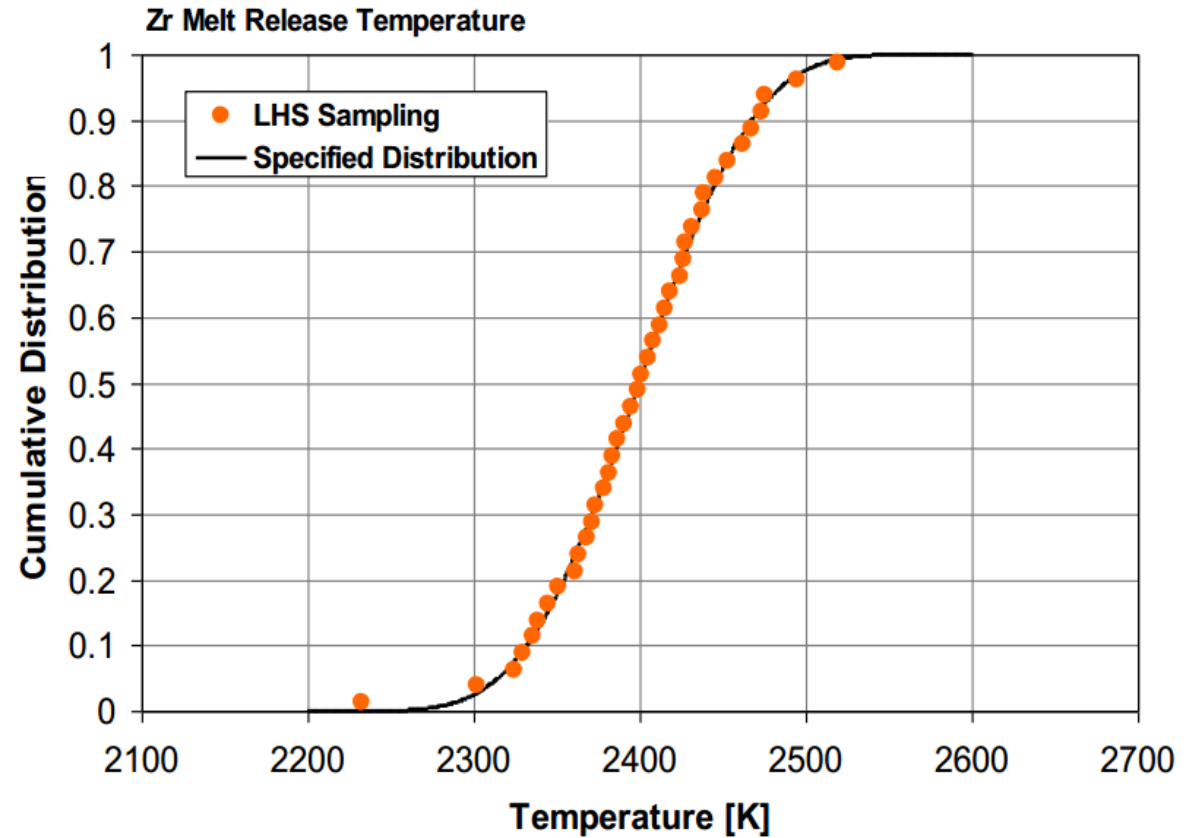
Confidence Level (%)	Sample Size to span p =			
	0.9	0.95	0.99	0.999
90	37	76	388	3888
95	46	93	473	4742
99	64	130	661	6635
99.9	88	180	919	9228

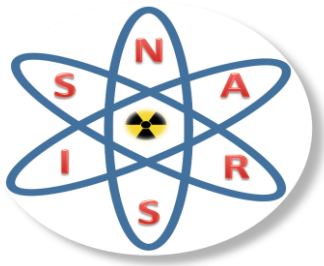


## Example – Sampling

### ➤ Sampling.

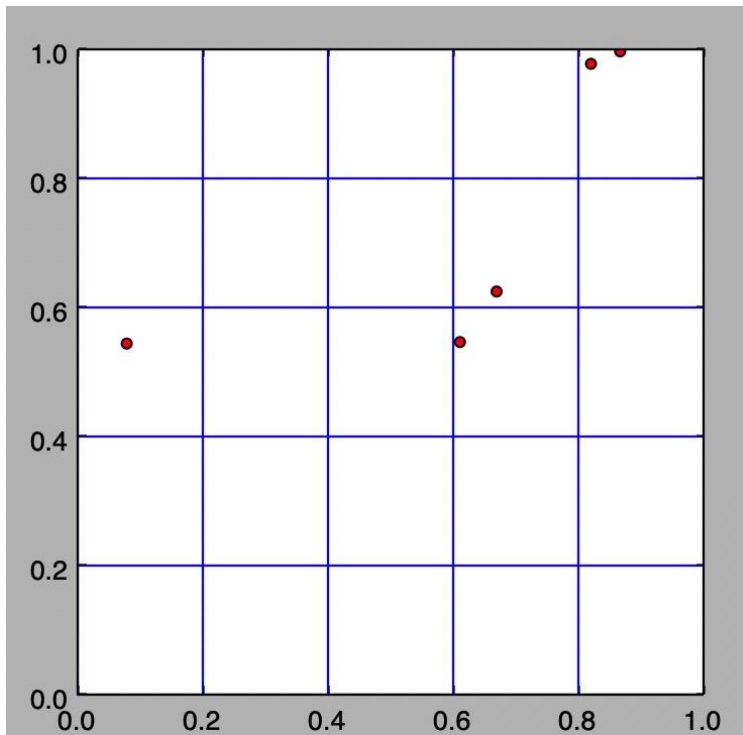
- ❑ SRS – Simple Random Sampling
- ❑ LHS – Latin Hypercube Sampling



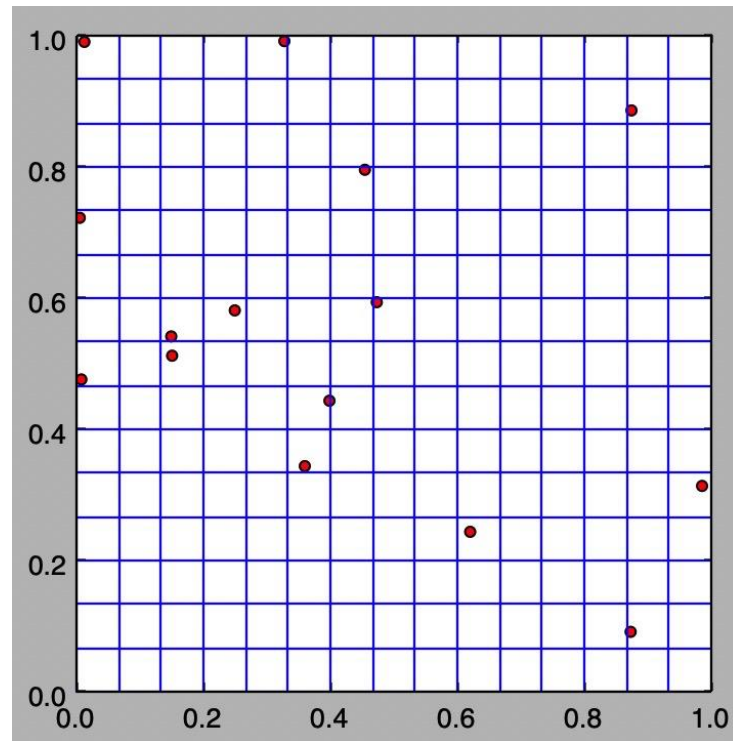


# Sampling: Standard Random Sampling

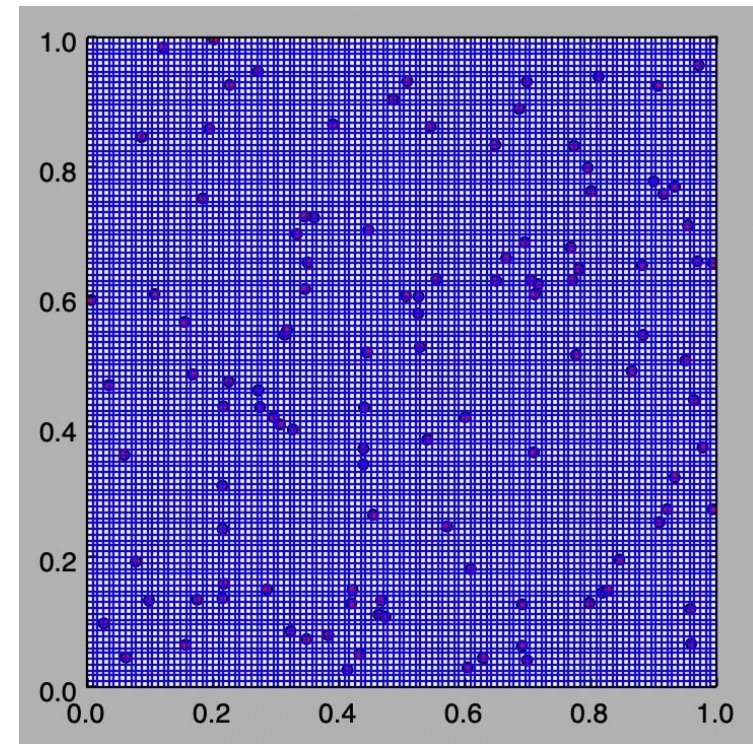
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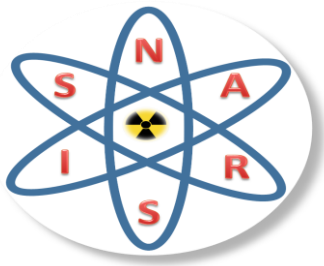


N=15



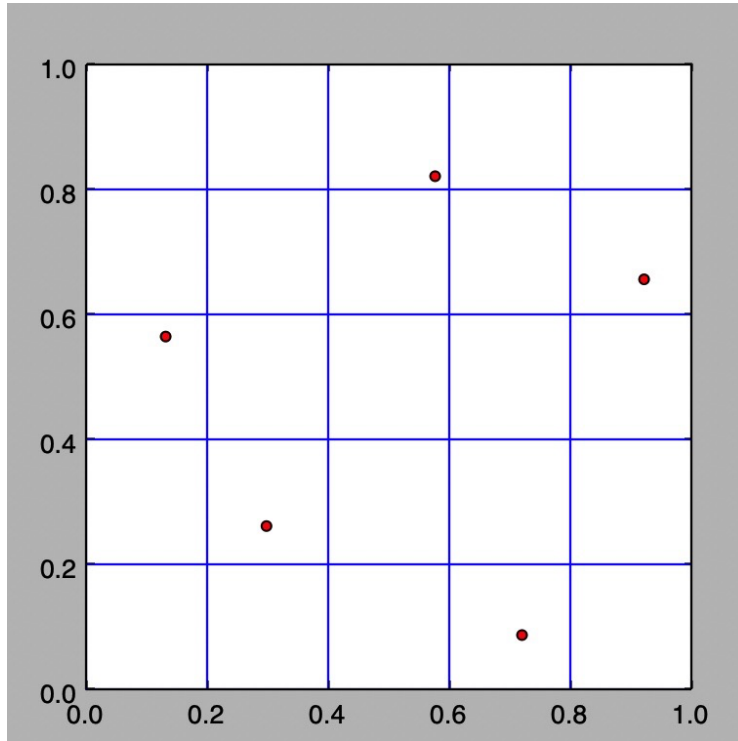
N=115



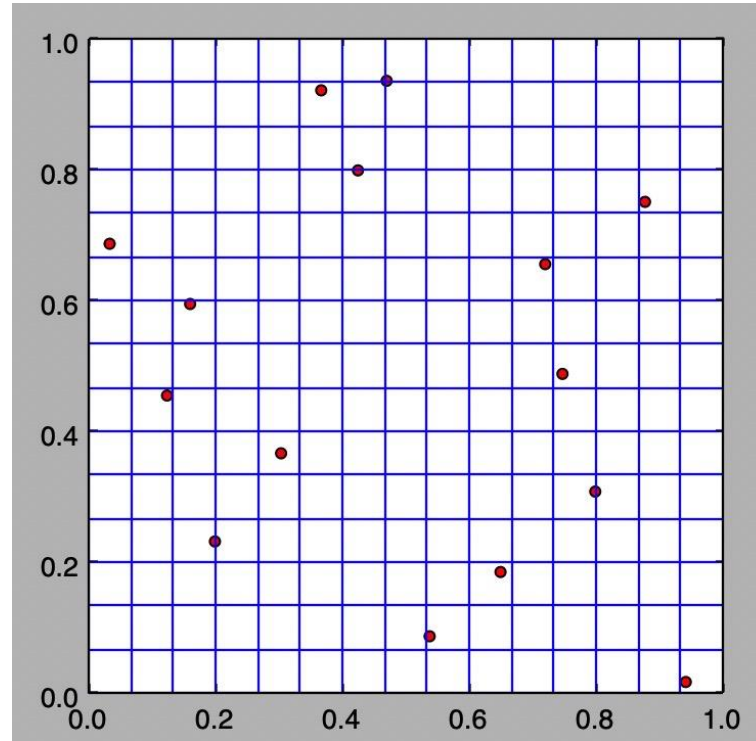


# Sampling: Latin Hypercube Sampling

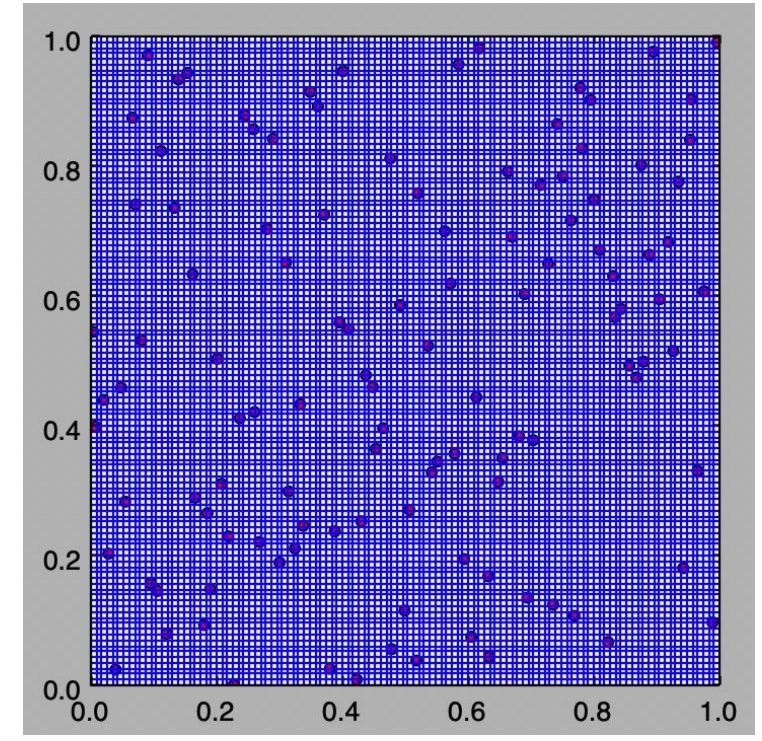
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N=15



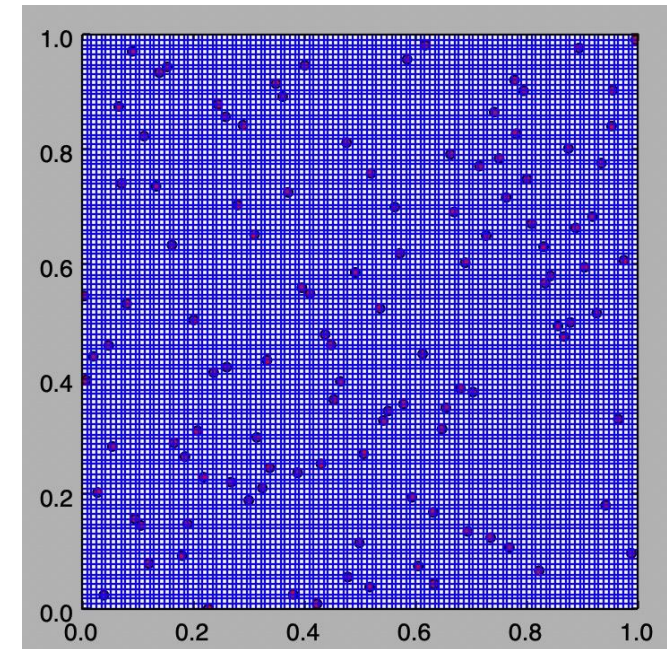
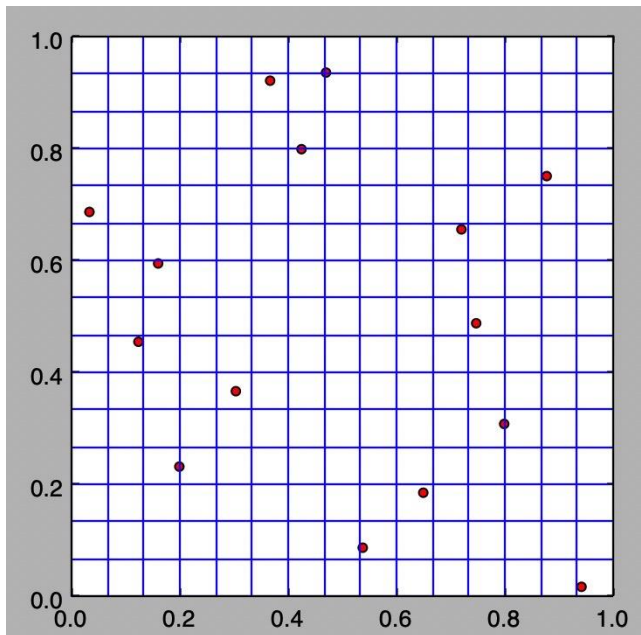
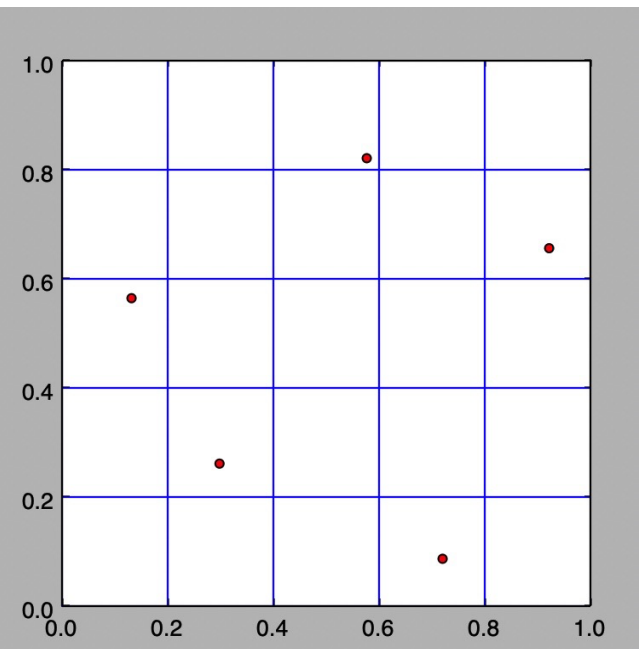
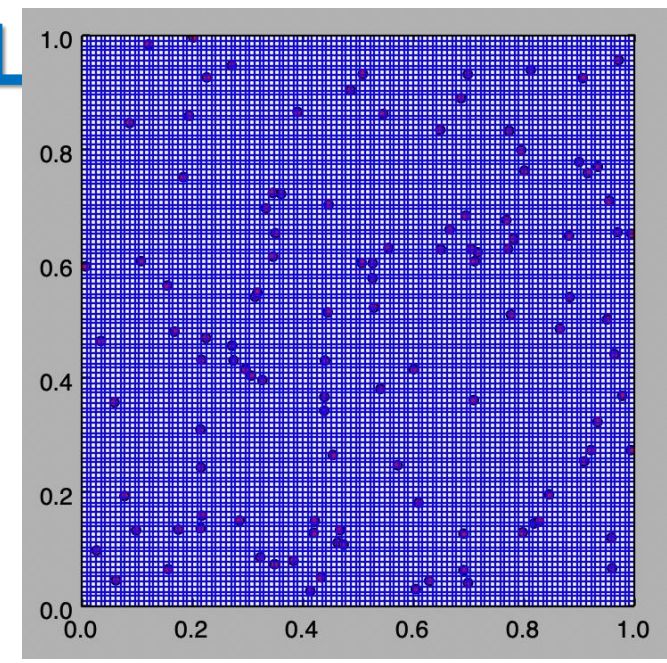
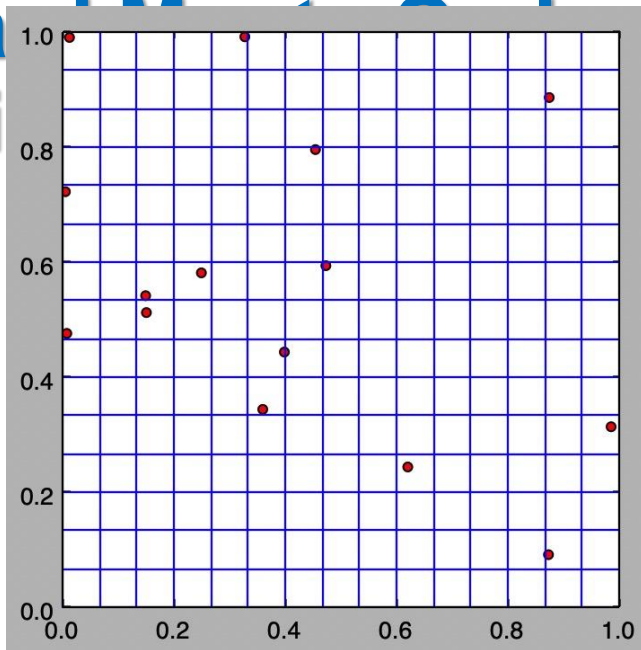
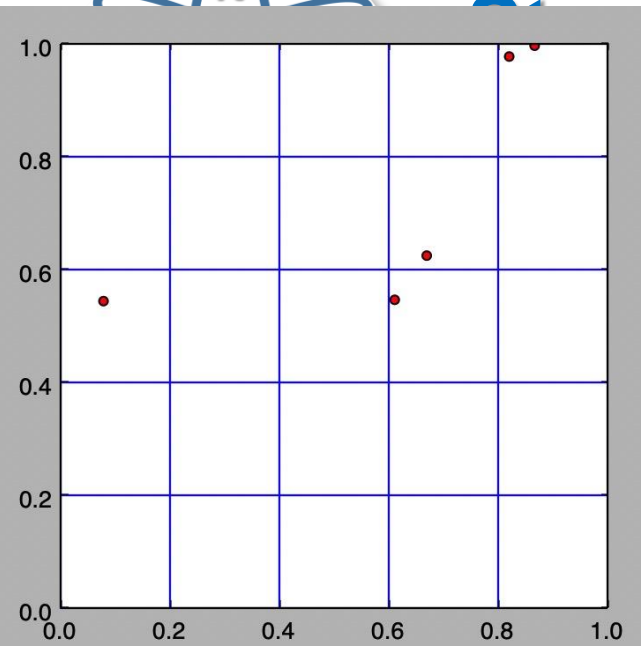
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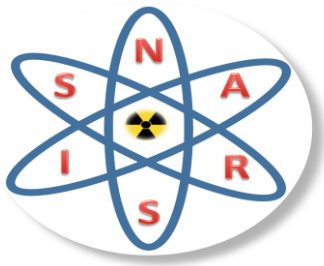




# data vs LHS

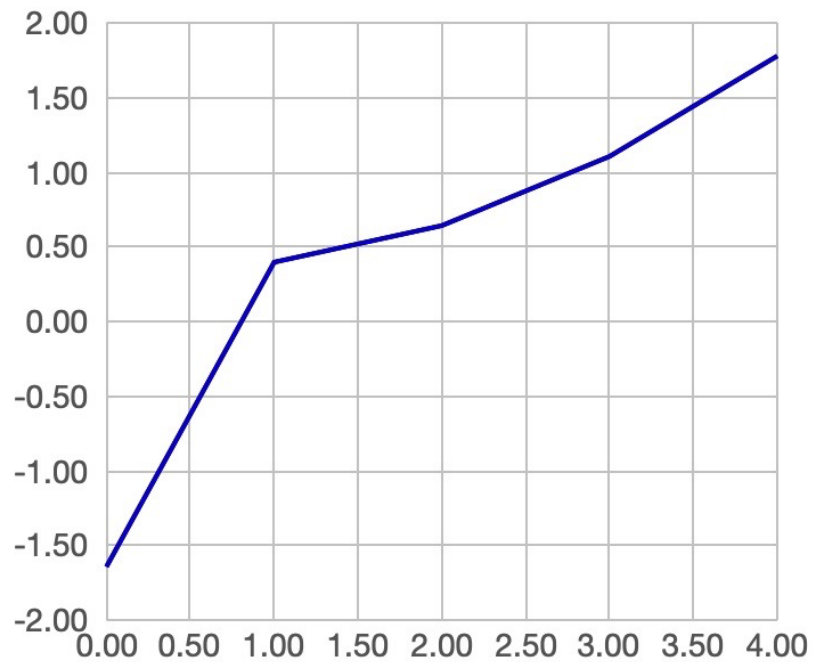
## pli



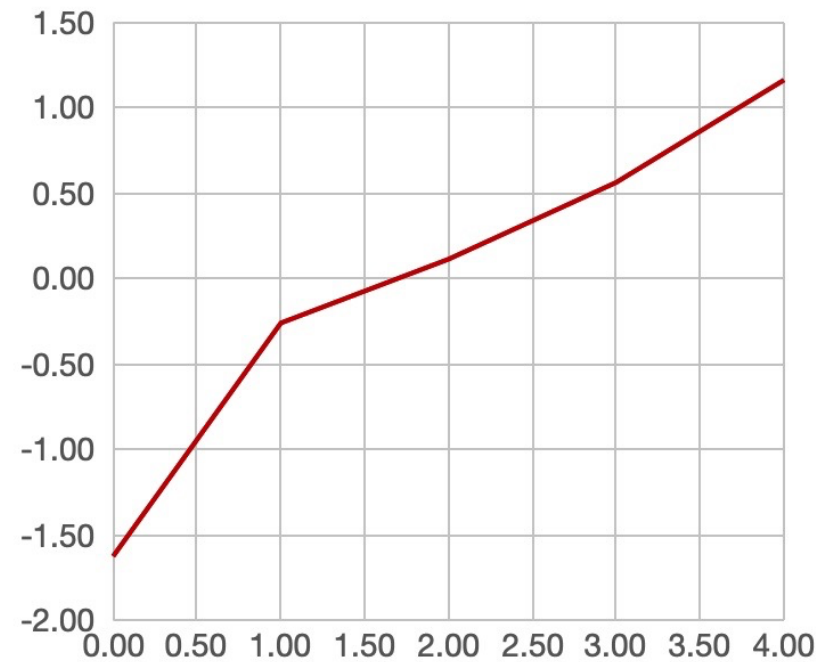


# SRS vs LHS

### Random Sampling



### Latin Hypercube Sampling

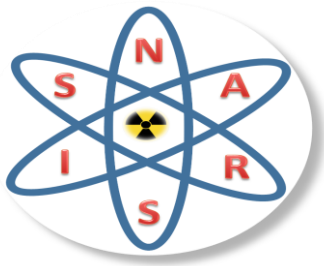


Count

5

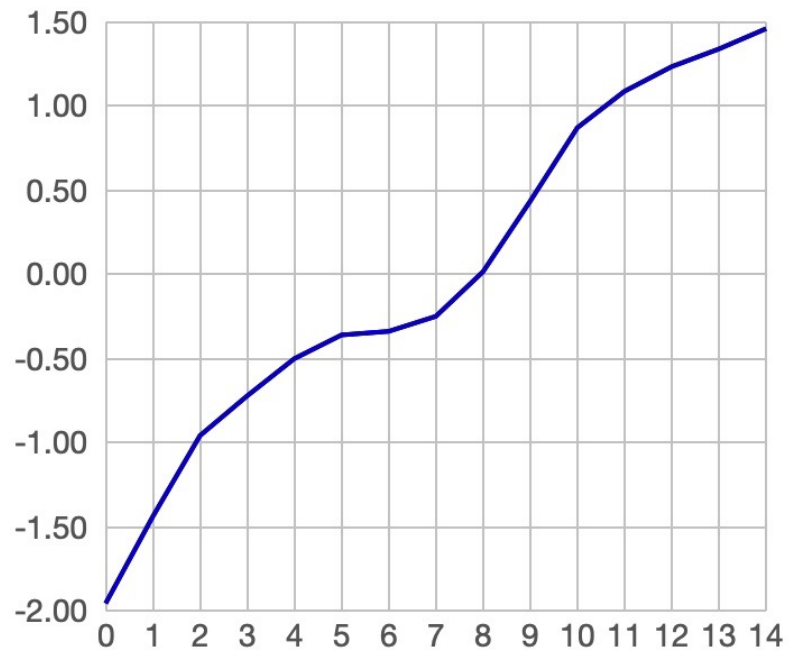
Redraw

Histogram

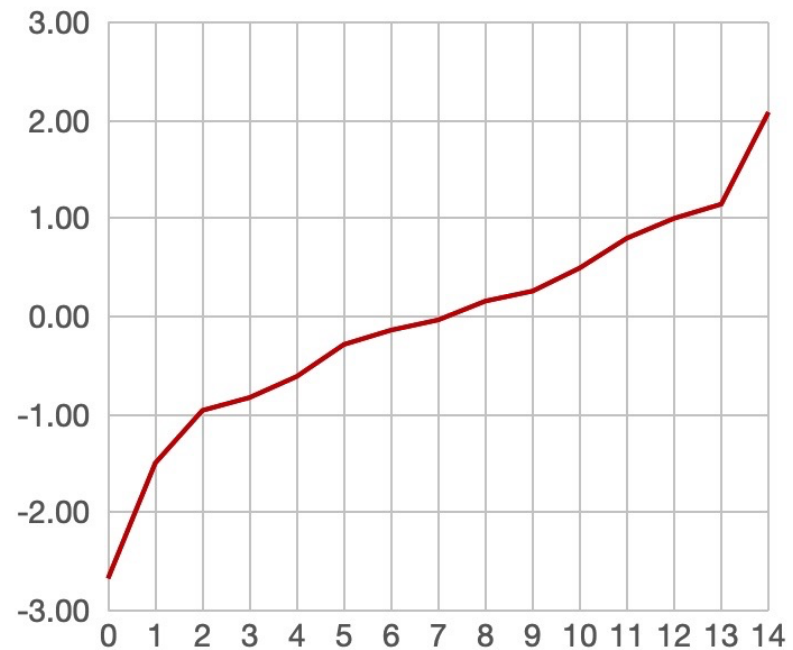


# SRS vs LHS

### Random Sampling



### Latin Hypercube Sampling



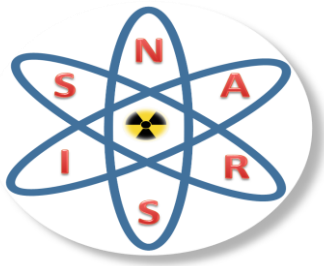
Count

15

Redraw

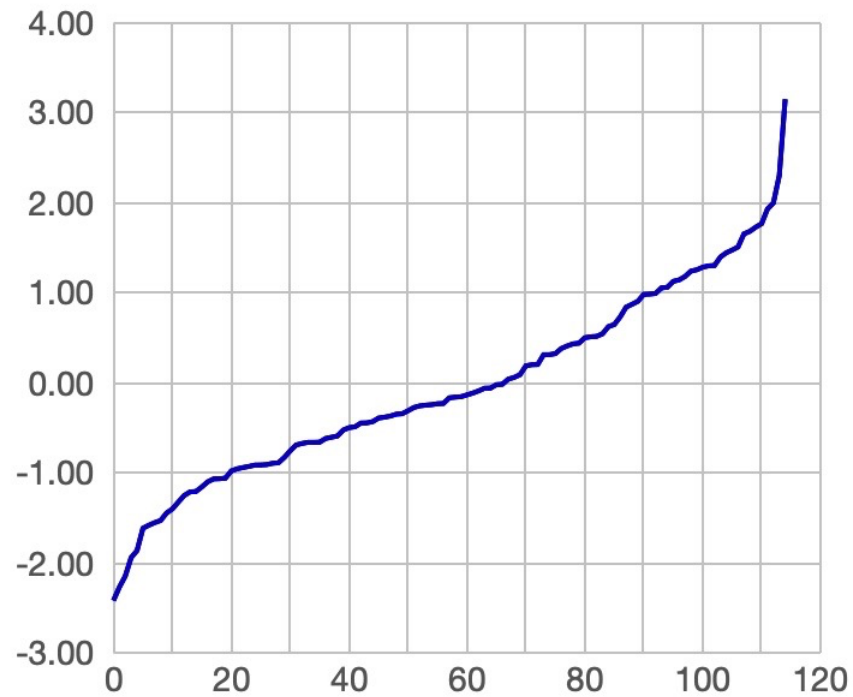
Histogram



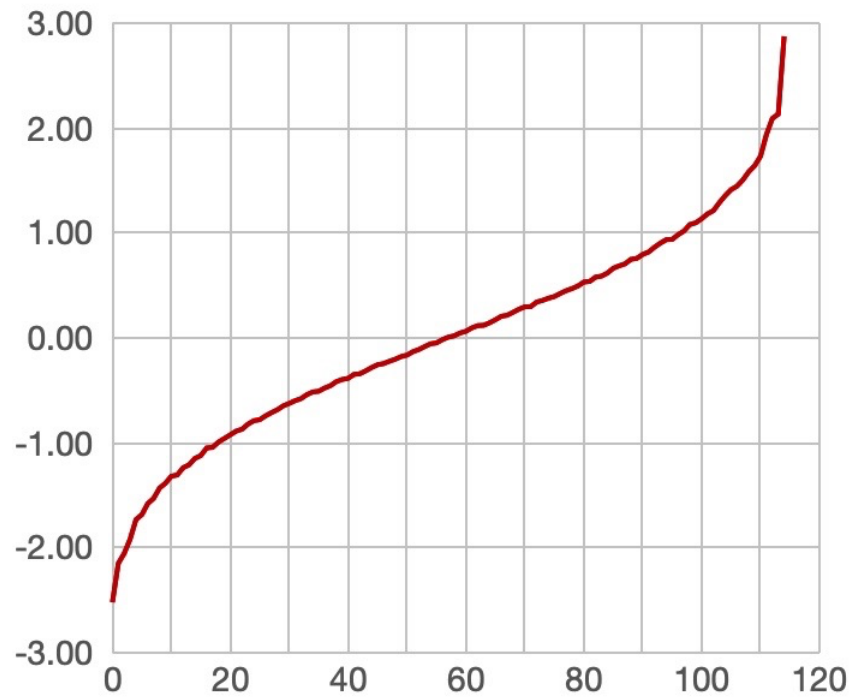


# SRS vs LHS

## Random Sampling



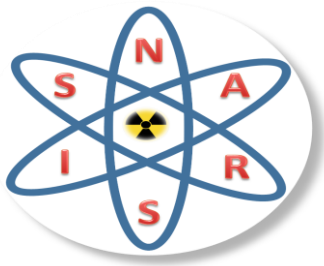
## Latin Hypercube Sampling



Count 115

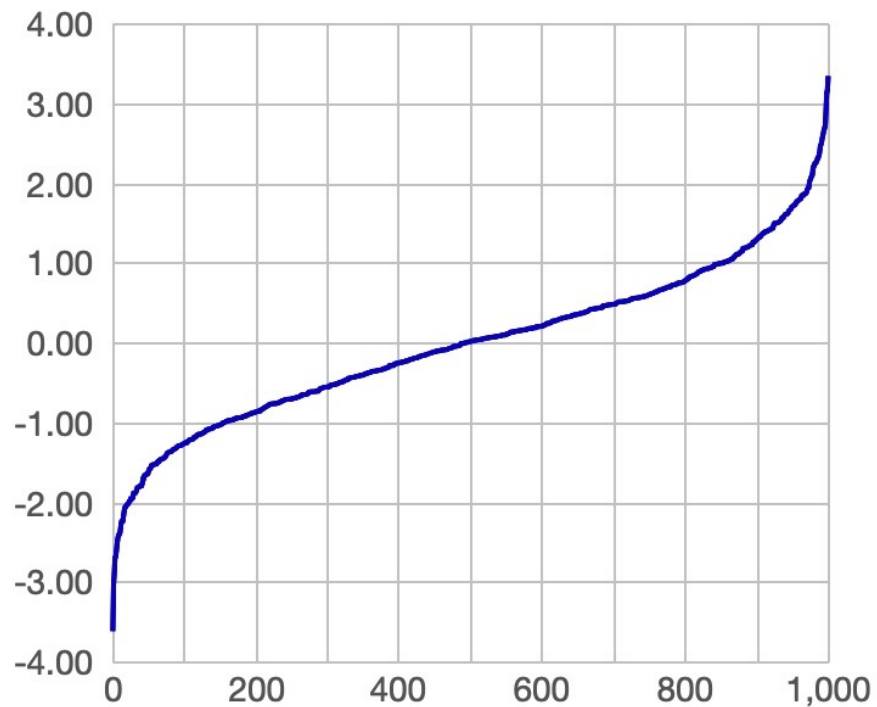
Redraw

Histogram

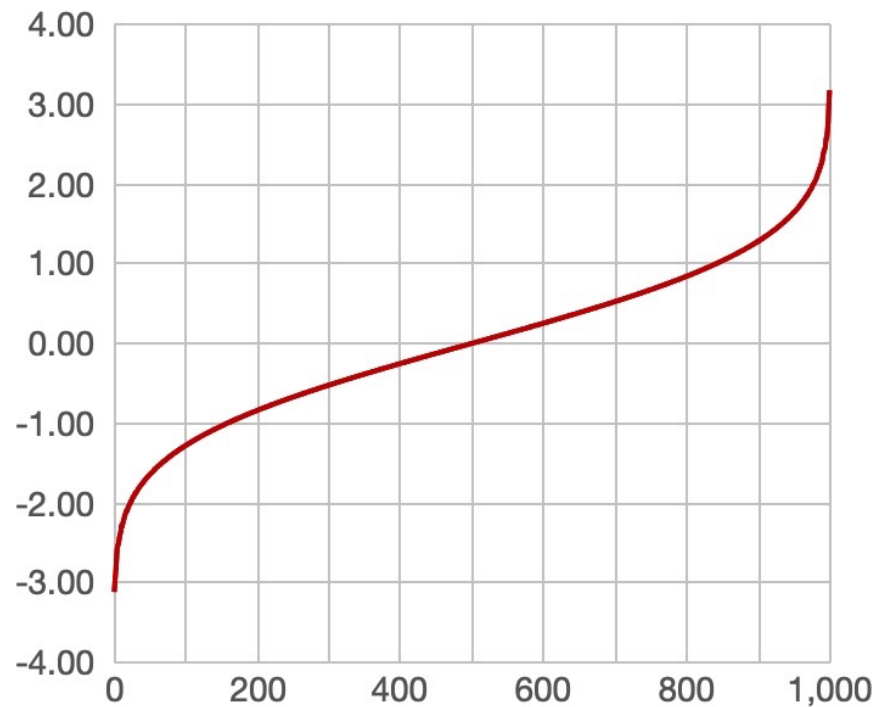


# SRS vs LHS

Random Sampling



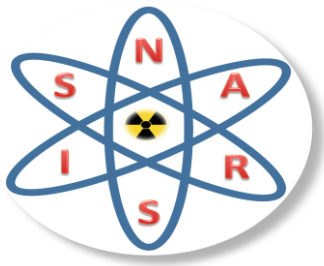
Latin Hypercube Sampling



Count 1000

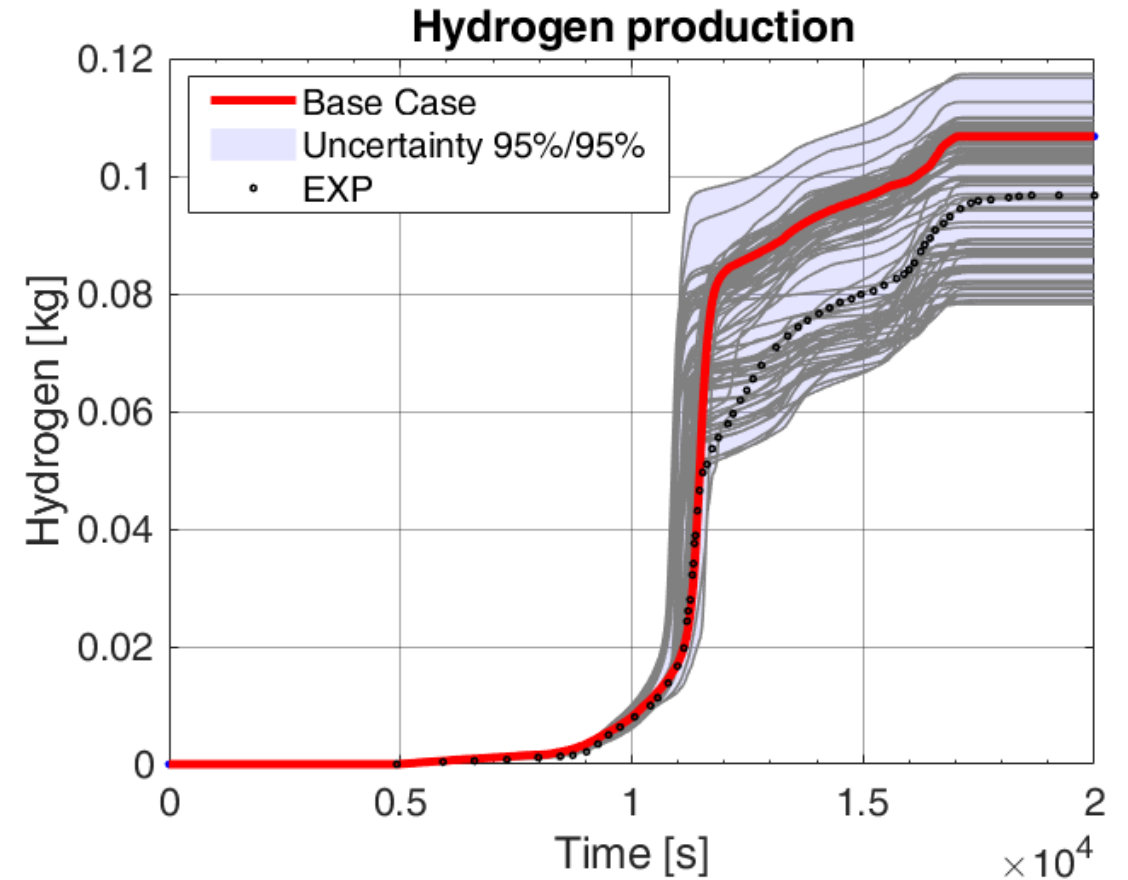
Redraw

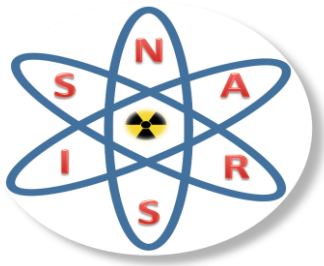
Histogram



## Example – Uncertainty

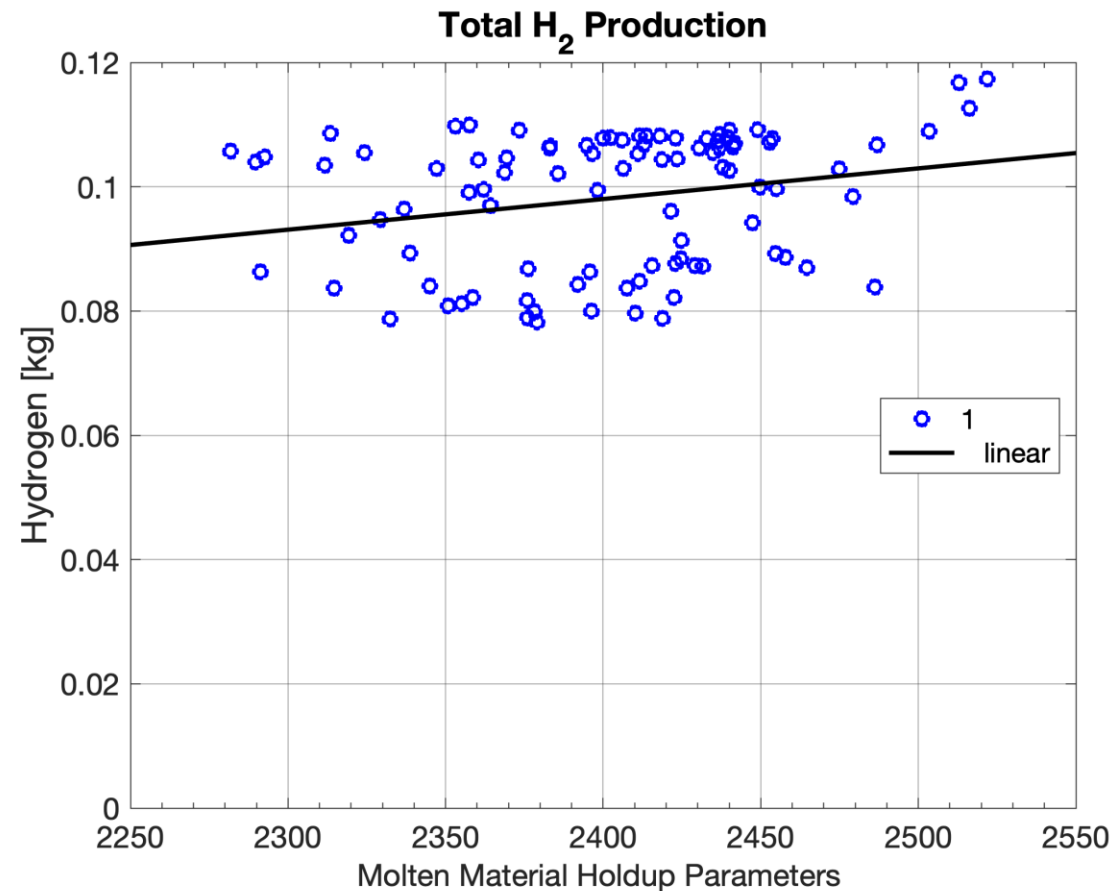
- **Generate models with sampled values.**
- **Perform Simulations**
- **Post-process results.**
- **Uncertainty analysis**
  - Confidence Intervals
  - Example: Limits from the sample represents the 95% confidence interval within which 95% of all the possible values lie.

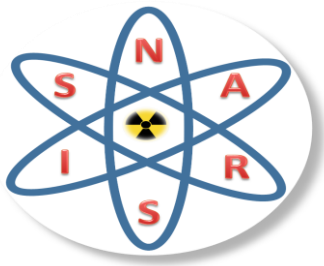




## Example – Sensitivity

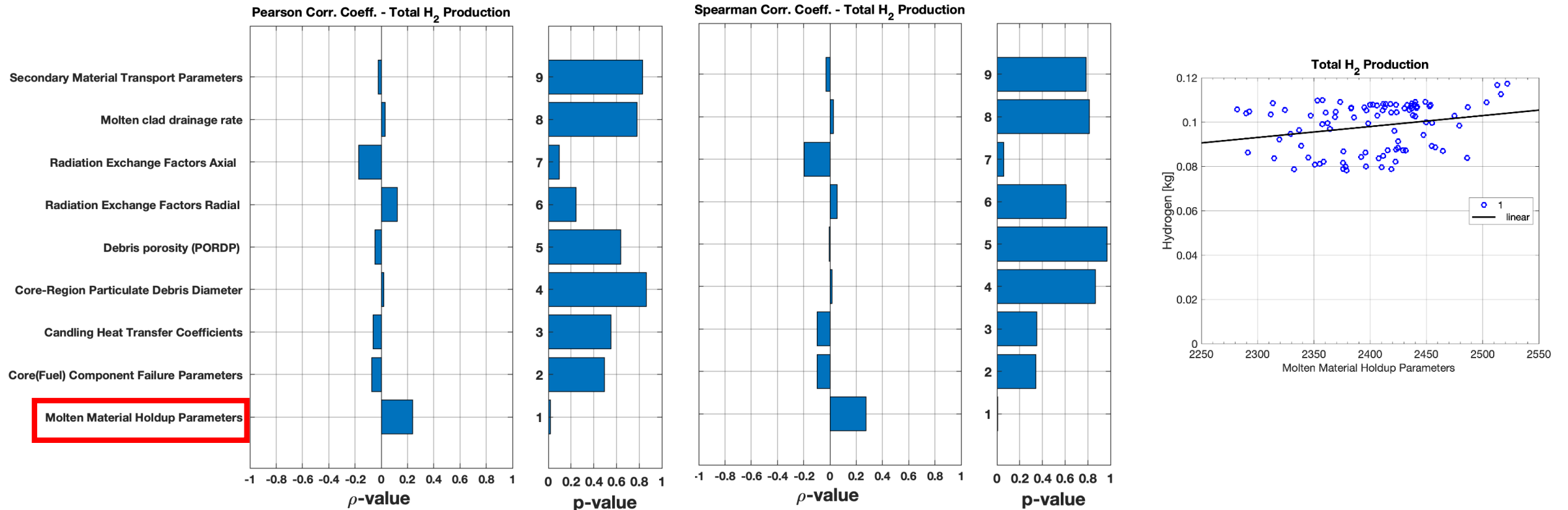
- **Linear regression is simple & popular in the literature**
- **Other more sophisticated are possible**

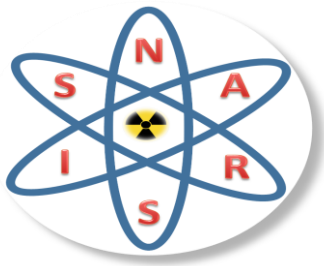




# Example – Sensitivity

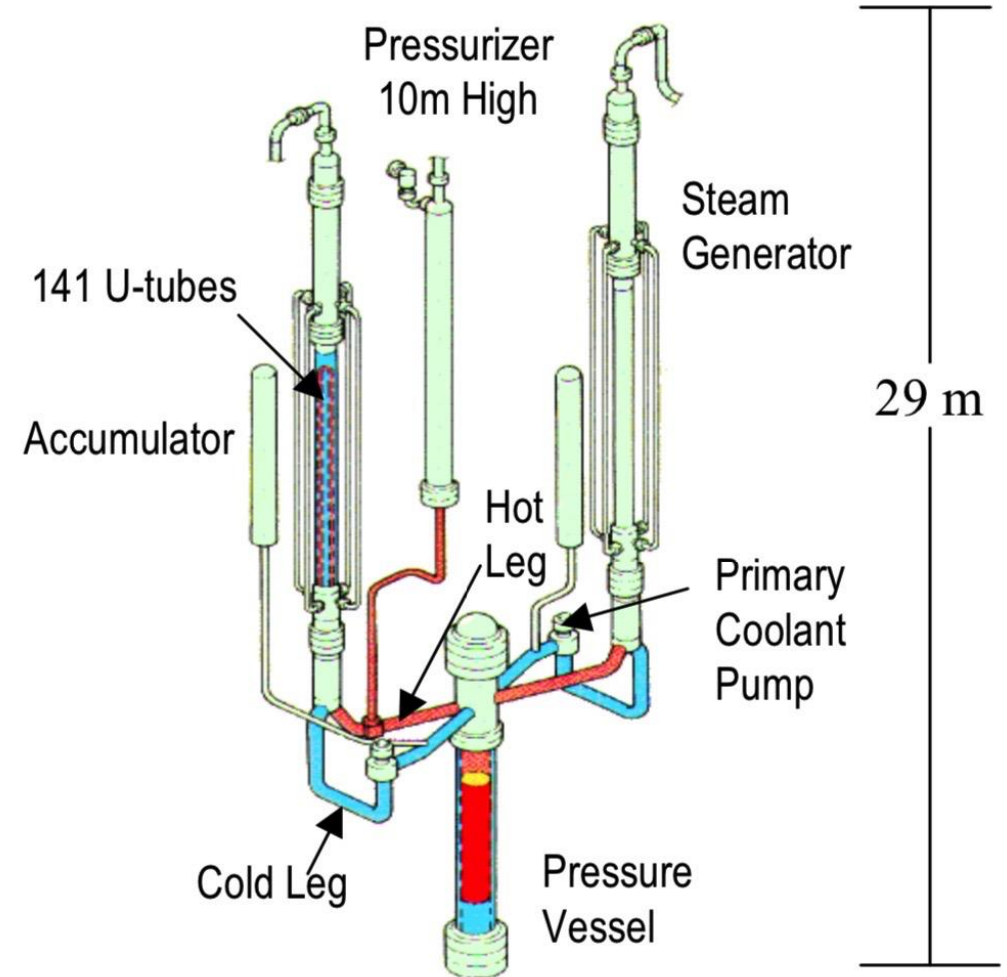
- Pearson (Linear) and Spearman (non-linear) are popular, other are also possible.
- For parameter #1, low p-value and „large” rho value indicates possible correlation (in Example it is weak).

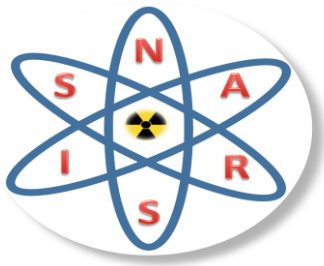




# Intermediate Break LOCA

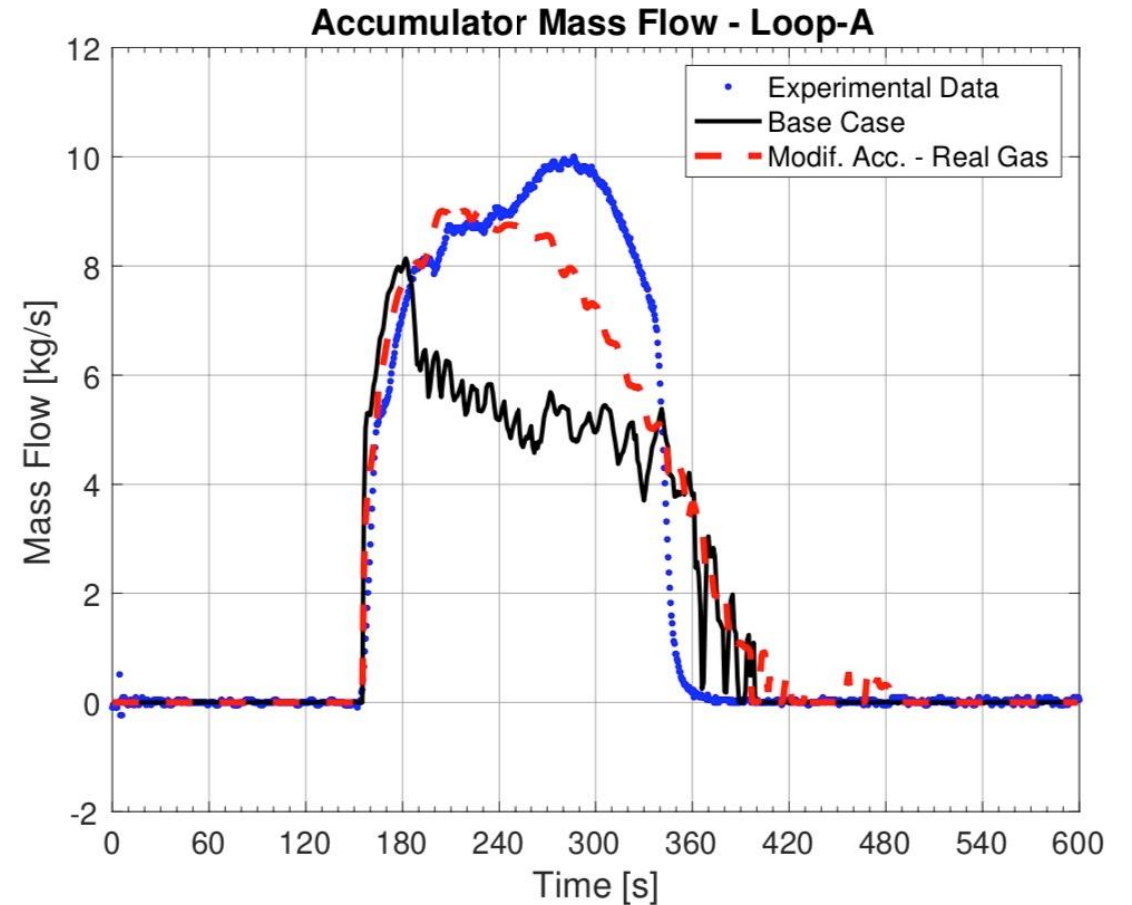
- Other example
- Large Scale Test Facility in Japan
- 13% break size of the cold leg

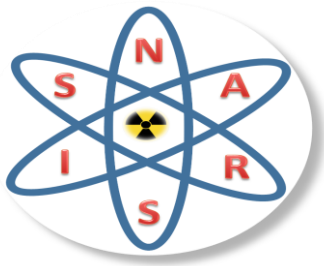




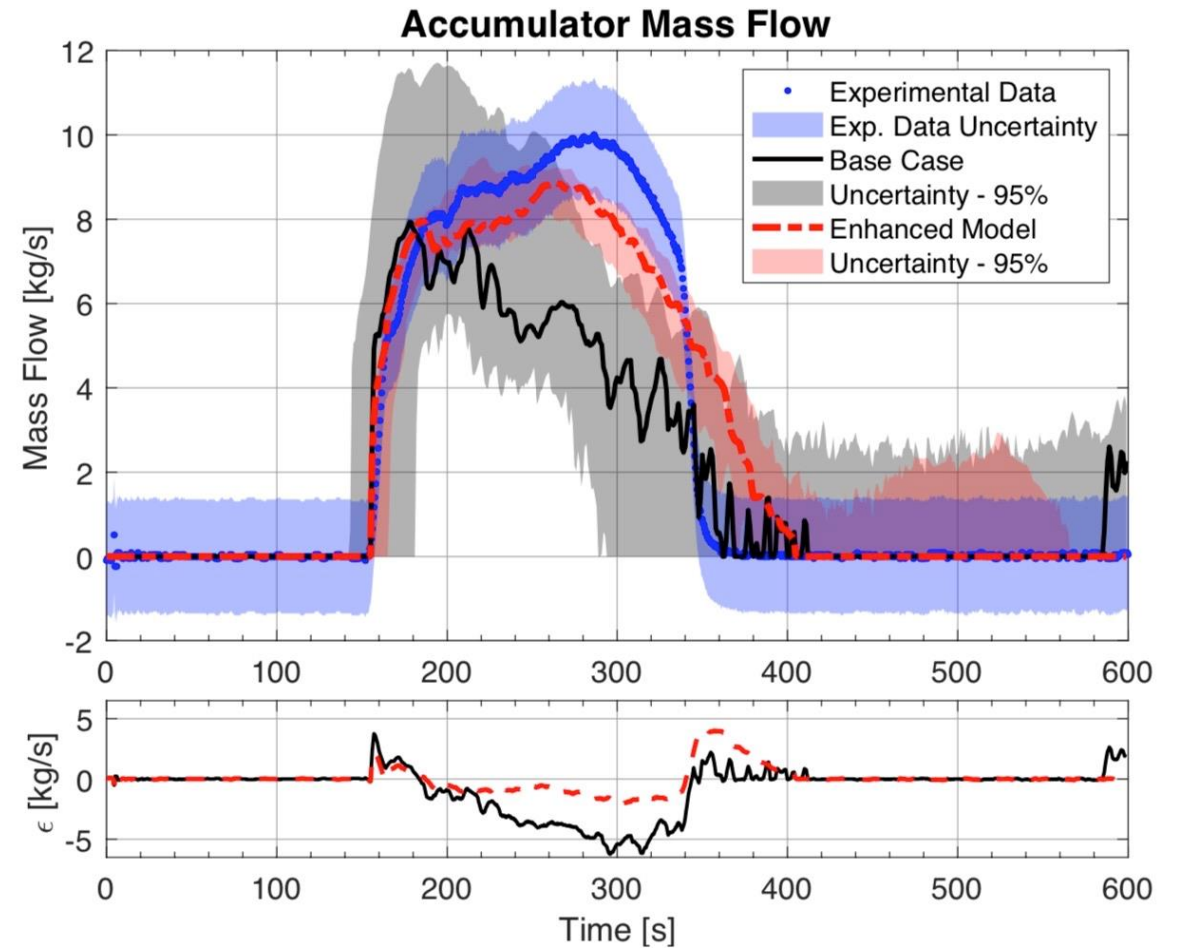
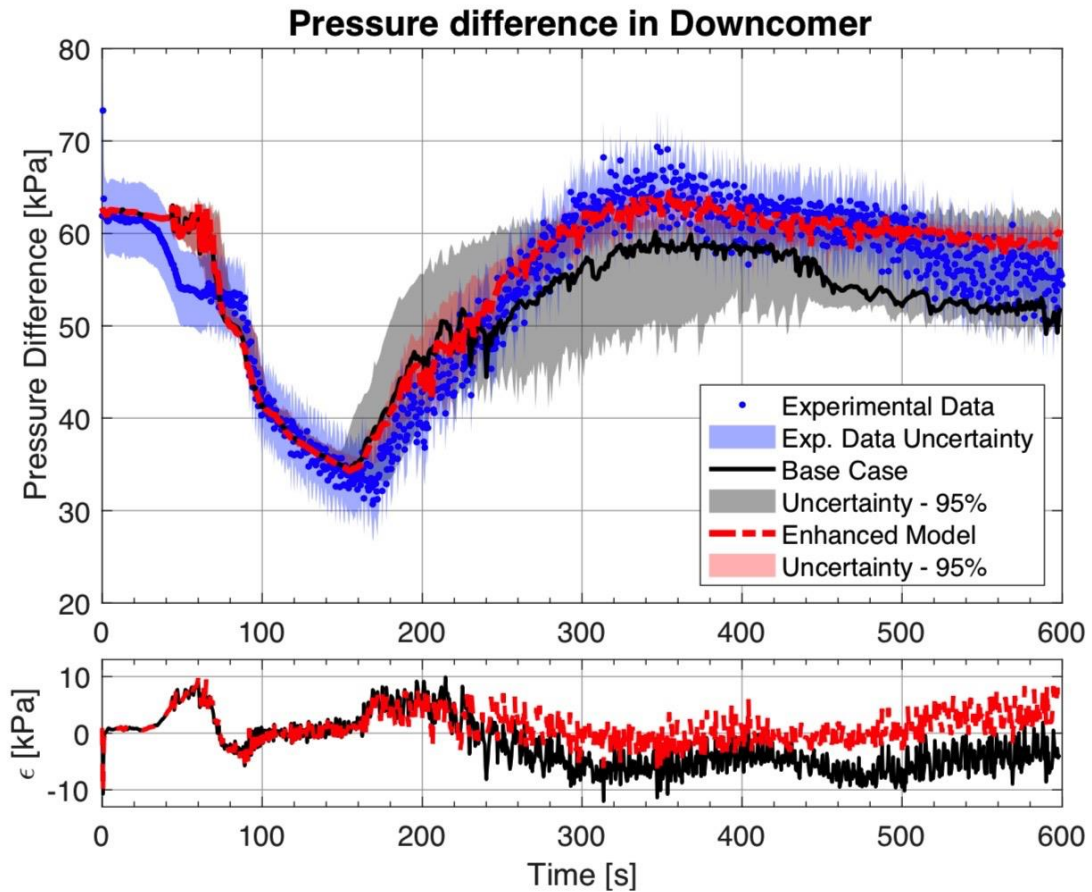
# Enhancement model of accumulator

- Standard model of accumulator (gas expansion) changed to more realistic, based on experimental data

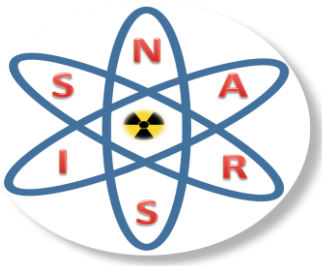




# Enhancement model of accumulator – how the uncertainties propagation changes

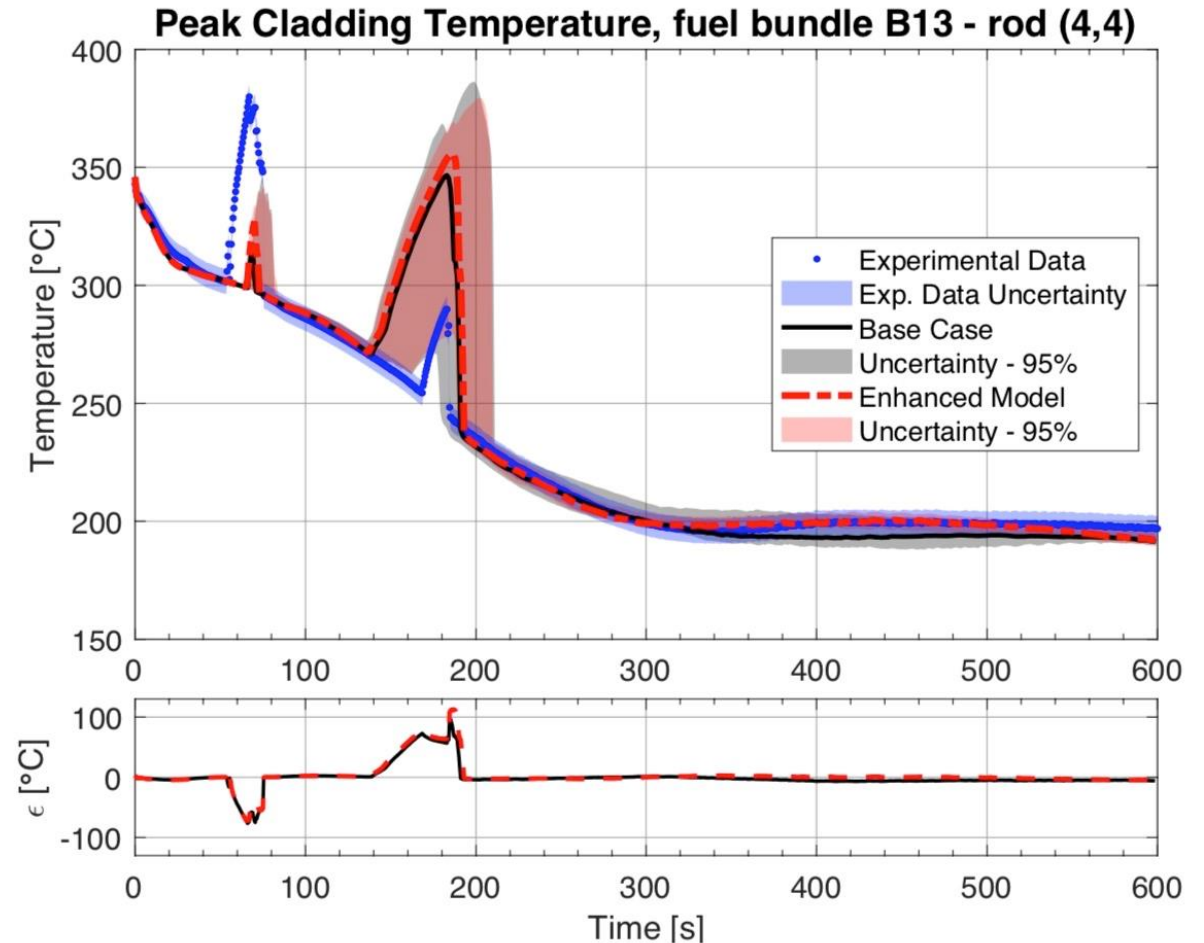


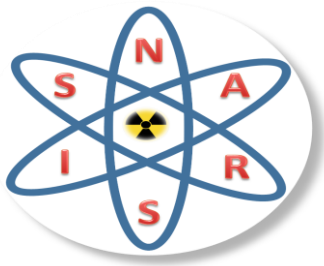




# How do we know that our model is correct ?

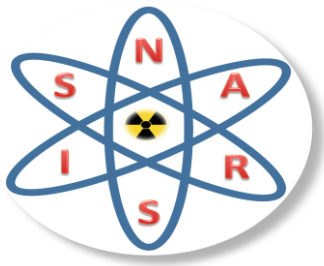
- After uncertainty modeling we can learn that we could commit errors in nodalization or boundary conditions, local measurements
- And what if there is no experimental data ? How can we be sure that even after uncertainties analysis we are on the safe side ?





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**Thank you for your attention**

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